

October 9, 1980

MEMORANDUM FOR: B. H. Grier, Director, Region I
 J. P. O'Reilly, Director, Region II
 J. G. Keppler, Director, Region III
 K. V. Seyfrit, Director, Region IV
 R. H. Engelken, Director, Region V

FROM: Norman C. Moseley, Director, Division of Reactor Operations
 Inspection, Office of Inspection and Enforcement

SUBJECT: IE INFORMATION NOTICE NO. 80-36: FAILURE OF STEAM GENERATOR
 SUPPORT BOLTING

The subject Information Notice is transmitted for issuance on October 10, 1980. The Information Notice should be issued to all nuclear power reactor facilities holding power reactor Operating Licenses or Construction Permits. Also enclosed is a draft copy of the transmittal letter for this purpose.

151

Norman C. Moseley, Director
 Division of Reactor Operations
 Inspection
 Office of Inspection and Enforcement

- Enclosures:
1. Draft transmittal letter
 2. IE Information Notice
 No. 80-36

CONTACT: R. A. Hermann, IE
 49-28180

| | | | | | |
|----------|--------------------|---------------------------|----------------------------------|---------------------------|--------------------|
| WPU: SM | ROI: IE <i>RAH</i> | <i>OBB for</i> ROI: IE | <i>NCM</i> ROI: IE <i>for</i> | <i>Carl Sw</i> RCI: IE | XOOS: <i>DT</i> |
| 10/1/80 | RAHermann | ELJordan | <i>for</i> NCMoseley | HDThornburg | DThompson |
| Job I(3) | 10/2/80 | 10/2/80 | 10/2/80 | 10/3/80 | <i>for</i> 10/3/80 |

(Draft letter to all nuclear power reactor facilities holding an Operating License or Construction Permit)

IE Information Notice No. 80-36: FAILURE OF STEAM GENERATOR SUPPORT BOLTING

Addressee:

This Information Notice is provided as an early notification of a possibly significant matter. It is expected that recipients will review the information for possible applicability to their facilities. No specific action or response is requested at this time. If further NRC evaluations so indicate, an IE Circular or Bulletin will be issued to recommend or request specific licensee actions. If you have questions regarding this matter, please contact the Director of the appropriate NRC Regional Office.

Sincerely,

Signature
(Regional Director)

Enclosures:

1. IE Information Notice
No. 80-36
2. List of Recently Issued
IE Information Notices

DUPLICATE

SSINS No.: 6835
Accession No.:
8006190085
IN 80-36

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

October 10, 1980

IE Information Notice No. 80-36: FAILURE OF STEAM GENERATOR SUPPORT BOLTING

Description of Circumstances:

On September 4, 1980, the NRC staff was informed by the Northern States Power Company that extensive cracking of the bolting for the supports attached to bottom of the steam generators on Prairie Island Unit 1 had occurred. The licensee initially discovered the bolting problem when two bolts failed during detensioning. The licensee was attempting to remove the pre-load on the bolting (specified at 1400 ft-lbs) to minimize the potential for stress corrosion cracking of the high strength 1-1/2 in. dia. 250 ksi maraging steel bolting (Vascomax 250).

Ultrasonic examination of the support bolting revealed cracks or breaks in 47 of 48 support bolts for the two steam generators at the Prairie Island Unit 1 facility. The licensee subsequently shut down the Prairie Island Unit 2 facility and performed ultrasonic examinations of the same steam generator support bolting, that failed on Unit 1. The results of the UT examinations revealed cracks in three of the 48 steam generator support bolts. The licensee attempted to remove the cracked bolts from the steam generator supports. Two cracked bolts were successfully removed while the third failed during removal. Two bolts that were judged as defect free by the ultrasonic examination were also removed. Magnetic particle examinations confirmed the results of the ultrasonic examinations on the bolts which were removed. As part of the short term corrective actions taken, the licensee has unloaded and retensioned the bolting at approximately 50 to 100 ft.-lbs.

Inservice ultrasonic examination of the affected bolting had been performed previously. In the case of Unit 1 a significant number of the bolts were examined at the last refueling outage 14 months ago. No cracks were found at that time.

The susceptibility for stress corrosion cracking was identified in NUREG-0577 (for comment). The NUREG identifies those facilities which have utilized maraging steels for reactor coolant pump and steam generator support bolting. Further the NUREG recommends augmented inspections for the facilities included in Group I.

Licensees are encouraged to review the bolting materials used at their respective facilities and consider inspection of maraging steel bolting that are either pre-loaded or significantly loaded during operation.

This Information Notice is provided as an early notification of a possibly significant matter that is still under review by the NRC staff. It is expected that recipients will review the information for possible applicability to their facility. No specific action or response is requested at this time. If you have any questions regarding this matter, please contact the Director of the appropriate NRC Regional Office.

RECENTLY ISSUED
IE INFORMATION NOTICES

| Information Notice No. | Subject | Date of Issue | Issued to |
|------------------------|--|---------------|---|
| 80-35 | Leaking and dislodged Iodine-124 implant seeds | 10/10/80 | All categories G and G1 medical licensees |
| 80-34 | Boron dilution of reactor coolant during steam generator decontamination | 9/26/80 | All pressurized water reactor facilities holding power reactor OLs |
| 80-33 | Determination of teletherapy timer accuracy | 9/15/80 | All teletherapy (G3) licensees |
| 80-32 | Clarification of certain requirements for Exclusive-use shipments of radioactive materials | 8/12/80 | All NRC and agreement state licensees |
| 80-31 | Maloperation of Gould-Brown Boveri Type 480 volt type K-600S and K-DON 600S circuit breakers | 8/27/80 | All light water reactor facilities holding OLs or CPs |
| 80-30 | Potential for unacceptable interaction between the control rod drive scram function and non-essential control air at certain GE BWR facilities | 8/19/80 | All boiling water reactor facilities holding power reactor OLs or CPs. |
| 80-29 | Broken studs on Terry turbine steam inlet flange | 8/7/80 | All light water reactor facilities holding power reactor OLs or CPs* |
| Supplement to 80-06 | Notification of significant events at operating power reactor facilities | 7/29/80 | All holders of reactor and near-term OL applicants |
| 80-28 | Prompt reporting of required information to NRC | 6/13/80 | All applicants for and holders of nuclear power reactor CPs |
| 80-27 | Degradation of reactor coolant pump studs | 6/11/80 | All pressurized water reactor facilities holding power reactor OLs or CPs |

* Operating Licenses or Construction Permits