



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

YANKEE ATOMIC ELECTRIC COMPANY

DOCKET NO. 50-29

YANKEE NUCLEAR POWER STATION (YANKEE-ROWE)

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50  
License No. DPR-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Yankee Atomic Electric Company (the licensee) dated June 7, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-3 is hereby amended to read as follows:

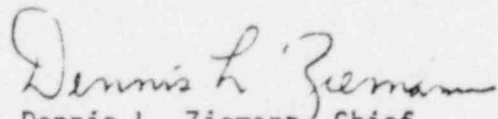
8011060 759

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 50, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 31, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 50

FACILITY OPERATING LICENSE NO. DPR-3

DOCKET NO. 50-29

Revise Appendix A Technical Specifications by removing the following pages and inserting the enclosed pages. The revised pages contain the captioned amendment number and vertical lines indicating the area of change. Overleaf pages are included for document completeness.

REMOVE

3/4 4-7  
3/4 10-3  
6-21

INSERT

3/4 4-7  
3/4 10-3\*  
6-21\*

\*These pages are included for the purposes of correcting clerical and administrative errors which occurred inadvertently during the issuance of Amendment No. 49, dated May 30, 1978.

MAIN COOLANT SYSTEM

PRESSURIZER

LIMITING CONDITION FOR OPERATION

---

3.4.4 The pressurizer shall be OPERABLE with a steam bubble.

APPLICABILITY: MODES 1 and 2

ACTION:

With the pressurizer inoperable, be in at least HOT STANDBY with the reactor trip breakers open within 6 hours.

SURVEILLANCE REQUIREMENTS

---

4.4.4 No additional requirements other than those specified in accordance with 10 CFR 50.55a(g).

MAIN COOLANT SYSTEM

3/4.4.5 MAIN COOLANT SYSTEM LEAKAGE

LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

---

3.4.5.1 The following Main Coolant System leakage detection systems shall be OPERABLE:

- a. The containment atmosphere particulate radioactivity monitoring system,
- b. The containment drain tank level monitoring system.
- c. The incore detection system thimble leak alarm system.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With the above required radioactivity monitoring leakage detection system inoperable, operation may continue for up to 7 days provided:
  1. Main Coolant System water inventory balance is performed at least once per 24 hours.
  2. The other above required leakage detection systems are OPERABLE, and
  3. Appropriate grab samples are obtained and analyzed at least once per hour:otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With the containment drain tank level monitoring system inoperable, restore the inoperable system to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With the incore detection system thimble leak alarm system inoperable, restore the leak alarm system to OPERABLE status within 7 days or close all thimble isolation valves; restore the leak alarm system to OPERABLE status within 31 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

## SPECIAL TEST EXCEPTIONS

### PRESSURE/TEMPERATURE LIMITATION - REACTOR CRITICALITY

#### LIMITING CONDITION FOR OPERATION

3.10.3 The minimum temperature and pressure conditions for reactor criticality of Specification 3.4.8.1 may be suspended during low temperature PHYSICS TESTS provided:

- a. The THERMAL POWER does not exceed 2 percent of RATED THERMAL POWER,
- b. The reactor low setpoints trips on the three OPERABLE Power Range Nuclear Channels are set at  $\leq 25\%$  of RATED THERMAL POWER, and
- c. The Main Coolant System temperature and pressure are maintained  $\geq 250^\circ\text{F}$  and  $\geq 300$  psig, respectively.

APPLICABILITY: MODE 2.

#### ACTION:

- a. With the THERMAL POWER  $> 2$  percent of RATED THERMAL POWER, immediately open the reactor trip breakers.
- b. With the Main Coolant System temperature and pressure  $< 250^\circ\text{F}$  or  $< 300$  psig, immediately open the reactor trip breakers and restore the temperature-pressure to within its limit within 30 minutes; perform the analysis required by Specification 3.4.8.1 prior to the next reactor criticality.

#### SURVEILLANCE REQUIREMENTS

4.10.3.1 The Main Coolant System temperature and pressure shall be verified to be  $\geq 250^\circ\text{F}$  and  $\geq 300$  psig at least once per hour.

4.10.3.2 The THERMAL POWER shall be determined to be  $\leq 2\%$  of RATED THERMAL POWER at least once per hour.

4.10.3.3 Each Power Range Nuclear Channel shall be subjected to a CHANNEL FUNCTIONAL TEST within 12 hours prior to initiating low temperature PHYSICS TESTS.

PHYSICS TESTS

LIMITING CONDITION FOR OPERATION

3.10.4 The limitations of Specification 3.1.1.4, 3.1.3.1, 3.1.3.4, and 3.1.3.5, may be suspended during the performance of PHYSICS TESTS provided:

- a. The THERMAL POWER does not exceed 2% of RATED THERMAL POWER, and
- b. The reactor low setpoint trips on the three OPERABLE Power Range Nuclear Channels are set at  $\leq 25\%$  of RATED THERMAL POWER.

APPLICABILITY: MODE 2.

ACTION:

With the THERMAL POWER  $> 2\%$  of RATED THERMAL POWER, immediately open the reactor trip breakers.

SURVEILLANCE REQUIREMENTS

4.10.4.1 The THERMAL POWER shall be determined to be  $\leq 2\%$  of RATED THERMAL POWER at least once per hour during PHYSICS TESTS.

4.10.4.2 Each Power Range Nuclear Channel shall be subjected to a CHANNEL FUNCTIONAL TEST within 12 hours prior to initiating PHYSICS TESTS.

## ADMINISTRATIVE CONTROLS

- (d) Total dissolved gas radioactivity (in curies) and average concentration released to the unrestricted area.
  - (e) Total volume (in liters) of liquid waste released.
  - (f) Total volume (in liters) of dilution water used prior to release from the restricted area.
  - (g) Total gross radioactivity (in curies) by nuclide released based on representative isotopic analyses performed.
  - (h) Percent of Technical Specification limit for total radioactivity.
- (3) Solid Wastes
- (a) The total amount of solid waste shipped (in cubic feet).
  - (b) The total estimated radioactivity (in curies) involved.
  - (c) Disposition including date and destination.

6.9.6 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Inservice Inspection Program Reviews, Specification 4.4.9.1.
- b. ECCS Actuation, Specifications 3.5.2 and 3.5.3.
- c. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.3.
- d. Sealed Source leakage in excess of limits, Specification 4.7.6.3.
- e. Radioactive Solid Waste Disposal, Specification 3.7.7.1.
- f. Fire Detection Instrumentation, Specification 3.3.3.4.
- g. Fire Suppression Systems, Specifications 3.7.10.1, 3.7.10.2 and 3.7.10.3.
- h. Environmental Monitoring Program, Specifications 3.7.8.



## ADMINISTRATIVE CONTROLS

### 6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspection, repair and replacement of principal items of equipment related to nuclear safety.
- c. All REPORTABLE OCCURRENCE reports submitted to the COMMISSION.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source leak tests and results.
- i. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Hazards Summary Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components identified in Table 5.7-1.