UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Docket No. 50-364

SEP 25 1980

APPLICANT: Alabama Power Company

FACILITY: Joseph M. Farley Nuclear Plant Unit No. 2

SUBJECT: SUMMARY OF JUNE 24, 1980 MEETING REGARDING REVIEW OF OPERATING LICENSE APPLICATION

The staff met with the applicant at its office in Bethesda, Md., to discuss outstanding review areas for the Farley Unit No. 2 plant operating license review. Enclosure 1 is the status of review of Non-TMI areas used as a handout for the meeting. Enclosure 2 is a list of attendees.

Mr. R. P. McDonald, Vice President, Nuclear Generation for Alabama Power Company (APCo), described his company's plans for completing Farley Unit 2 by September 1, 1980. Applicant was familiar with the TMI review requirements in NUREG-0694 and had prepared responses, dated June 20, to these requirements, based on SECY-80-230, a May 2, 1980 paper for the Commission regarding TMI-2 Action Plan.

Mr. V. Moore and Mr. G. Rivenbark described the new requirements for shift staffing, resulting from lessons learned from the Three Mile Island accident. Mr. G. Hairston, Plant Manager, said APCo planned to have a shift supervisor on each unit plus a senior reactor operator for the plant and one reactor operator for each unit plus a relief operator.

Mr. R. Bosnak mentioned the need to respond to IE Bulletin 79-02, Revision 1, "Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts", June 21, 1979 and to IE Bulletin 79-14, "Seismic Analyses for As-Built Safety-Related Piping Systems", June 2, 1979. He stated that there was no design problems, but that the applicant should work with I&E to complete the actions required by the bulletins. Mr. McDonald said the applicant was working with Region II, I&E, to complete the action items on these bulletins. Mr. Bosnak also mentioned that the staff was reviewing the valves separating high pressure from low pressure systems.

Mr. L. Phillips said that a reactor vessel level instrument must be developed and installed on Farley Unit 2. Mr. Oliver Kingsley, Manager, Nuclear Engineering and Technical Support , APCo, said that applicant was not satisfied that the Westinghouse proposed system, using pressure difference between the top and bottom of the reactor vessel, would be accurate enough. He said that applicant was considering another system, using neutron detectors.

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Mr. O. Kingsley said that selected emergency operating procedures would be filed for NRR review, in accordance with TMI requirements.

Mr. V. Noonan stated the staff's requirements for ports in the steam generators above the top tube sheet to view clearances between tubes and support plate holes to detect potential denting of steam generator tubes. He stated that the staff is reviewing the Westinghouse program to investigate causes of tube leaks in the first row of tubes and that no further action by applicant is required at this time.

Mr. Noonan also described the review of safety related electrical equipment to show conformance to NUREG-0588. Prior to issuing a full power license, the applicant's report will be given a mini-review to find major deficiencies, if any. The staff's SER on Environmental Qualification will be issued February 1, 1981, and all equipment will be upgraded by June 30, 1982. Regional meetings will be held in July 1981 to clarify this review.

L. L. Kintner

L. L. Kintner, Project Manager
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Status of Non-TMI Review Areas
2. List of Attendees

cc w/enclosures:
See next page

Mr. F. L. Clayton, Jr., Senior Vice
President
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NRC Resident Inspector
P. O. Box 1814
Dothan, Alabama 36302

ENCLOSURE 1
June 19, 1980

STATUS OF NON-TMI REVIEW AREAS FARLEY 2

1. Anticipated Transients Without Scram (ATWS)

   Emergency procedure has been developed. LPM has requested copy from Farley resident inspector.

   Staff June 13 letter requested procedure to be submitted by June 20.

2. Containment Sump Performance

   Safety evaluation accepts performance subject to development of "house keeping" program, reevaluation of equipment insulation in containment, development of procedures to detect and correct inadequate ECCS flow.

3. Control Rod Guide Tube Wear

   A surveillance program to demonstrate acceptable wear of control rod guide thimble tubes has been established for Salem Unit 1. This issue is resolved for Farley 2.

4. Environmental Qualification of Barton Pressure Transmitters

   Farley-2 used Barton Lot 2 pressure transmitters. Staff is evaluating test results.

5. Degraded Grid Voltage

   Staff is evaluating applicant's responses

6. Secondary Water Chemistry

   Farley-2 water chemistry program is acceptable and will be referenced in a license condition. Applicant is responding to staff positions on condenser tube plugging requirements, steam generator inspection ports, and plugging row 1 tubes in the steam generator.

   6a. Control Rod Guide Tube Support Pins

   Staff has required that support pins be replaced with new pins that have been given a heat treatment to minimize stress corrosion cracking.

   Applicant has agreed to do this

7. Preservice Inspection Program

   Staff is evaluating the PSI program and is awaiting additional information.

8. Appendix G&H Exemptions

   Staff has prepared a safety evaluation, but required a substantial amount of additional information to complete its review.

9. Seismic Qualification Review

The NRC review team has selected equipment to be reviewed and plans an inspection of equipment and audit of qualification records for some of this equipment during the week of July 7.

10. Inservice Testing of Pumps and Valves

Interim approval of the test plan is given for the portion of the initial 120 month inspection interval required to complete the final review.

11. Appendix J Exemptions

Staff has reviewed containment leakage testing methods and concluded they are acceptable, except for ECCS systems that recirculate water from the containment sump. Staff has requested additional information on the leakage limits for these systems and the basis including radiological assessment.

12. Containment Overpressure Due to a Main Steamline Break

Staff agrees with applicant's finding that peak containment pressure is below containment design pressure assuming a LOCA with auxiliary feedwater at the pump runout flow rate.

13. Revised Radwaste Systems

Applicant plans to use the Farley 1 solid waste processing system for both units. Staff concludes that proposed Farley Plant solid processing and drum storage facilities are adequate for both units.

14. Non-Radiological Environmental Technical Specifications

Applicant has proposed specifications for environs around the Farley Plant. Staff is reviewing these specifications.

15. Revised Radiation Protection Features

Applicant has provided additional information regarding the spent fuel transfer tube, respiratory equipment, and radiation protection personnel at the plant. Staff is reviewing.

16. Revised Operating Personnel Requirements

Minor changes in operations personnel requirements have been recorded in Amendments to the FSAR, and are under review.

17. Revised Quality Assuance Program

Minor changes to the Quality Assurance Program have been found to be acceptable.

18. Shared Systems Between Unit 1 and Unit 2

Shared portions of the cooling water systems and the onsite emergency

electrical power systems (diesel generators) are under review to determine the acceptability of changes made since our previous safety evaluation in June, 1977.

19. Feedwater Pipe Cracking

Applicant has been requested to inspect the feedwater piping of Farley 2 following hot functional testing, per I&E Bulletin 79-13 Rev. 2.

20. Confirmatory Piping Analysis

Staff Consultants are performing an independent code analysis of the "A" Main Steam Line inside containment to confirm that stresses are within code allowable valves.

21. Q-List for Quality Assurance Program

The staff is reassessing the list of safety related equipment required to be included in the operations quality assurance program.

22. Containment Purge Valves

Staff is evaluating the mechanical operability and containment functions of containment purge valves. A position regarding operability was transmitted to the applicant.

23. Revised Startup Test Program

Minor changes in the program description in the FSAR are under review.

24. Reduced Startup Physics Tests

Applicant has proposed reduced physics tests, similar to those proposed by the applicant for North Anna. Staff is reviewing these.

25. Environmental Qualification of Equipment

Staff requested applicant to review equipment to show conformance to NUREG-0588.

ENCLOSURE 2

LIST OF ATTENDEES

NRC MEETING WITH ALABAMA POWER COMPANY
JUNE 24, 1980

Nuclear Regulatory Commission

L. L. Kintner
V. A. Moore
G. W. Rivenbark
T. L. Huang
L. Phillips
A. Schwencer
R. L. Tedesco
D. G. Eisenhut
V. Noonan
R. Bosnak
R. L. Ballard

Southern Company Services, Inc.

T. M. Milton
O. Batum
R. J. George

Alabama Power Company

W. G. Hairston
R. L. George
R. P. McDonald
O. D. Kingsley
F. L. Clayton

Westinghouse

R. J. Sero
J. L. Vota
D. B. Pierce
T. R. Puryear

Bechtel

A. A. Vreil