



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report Nos. 50-324/80-32 and 50-325/80-35

Licensee: Carolina Power and Light Company
411 Fayetteville Street
Raleigh, NC 27602

Facility Name: Brunswick Units 1 and 2

Docket Nos. 50-324 and 50-325

License Nos. DPR-71 and DPR-62

Inspection at Brunswick Units 1 and 2 near Southport, NC

Inspector: J. E. Guzts

9/24/80
Date Signed

Approved by: P. J. Kellogg, Section Chief, RONS Branch

9/24/80
Date Signed

SUMMARY

Inspection on August 1-31, 1980

Areas Inspected

This routine, announced inspection involved 55 inspector-hours onsite in the areas of control room observations, review of plant logs, tours of vital areas, tours of physical security barriers, radiation protection, followup on IE Bulletin 80-17, review of licensee event reports (LERs) and independent inspection effort.

Unit 1 Areas Inspected

This routine inspection by the Resident Inspector involved 30 man-hours of inspection time onsite in the ten areas identified above.

Unit 1 Findings

Of the ten areas inspected no items of noncompliance or deviations were identified.

Unit 2 Areas Inspected

This routine inspection by the Resident Inspector involved 25 man-hours of inspection time onsite in the ten areas identified above.

Unit 2 Findings

Of the ten areas inspected no items of noncompliance or deviations were identified.

DETAILS

1. Persons Contacted

Licensee Employees

- *A. C. Tollison, Plant General Manager
- *J. M. Brown, Operations Manager
- *G. T. Milligan, Maintenance Manager
- *R. E. Morgan, Operations Manager
- *J. A. Padegett, QA and Safety Director
- *W. M. Tucker, Administrative and Technical Manager

Other licensee employees contacted included four technicians, ten operators, two mechanics, six security force members and two office personnel

*Attended exit interview.

2. Exit Interview

The inspection scope and findings were summarized on August 8 and 15, 1980 with those persons indicated in Paragraph 1 above.

3. Licensee Action on Previous Inspection Findings

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Operational Safety Verification

The inspector observed control room operations, reviewed applicable logs and conducted discussions with control room operators during the month of August. The inspector verified the operability of selected emergency systems, reviewed tagout records and verified proper return to service of affected components. Tours of vital areas including the reactor building and diesel buildings were conducted to observe plant equipment conditions, including potential fire hazards, fluid leaks, excessive vibrations and to verify that maintenance requests had been initiated for equipment in need of maintenance. The inspector, by observation and direct interview, verified that the physical security plan was being implemented in accordance with the station security plan.

The inspector observed plant housekeeping/cleanliness conditions and verified implementation of radiation protection controls.

No items of noncompliance or deviations were identified in these areas.

6. Licensee Event Reports Followup

Through direct observations, discussions with licensee personnel, and review of records, the following event reports were reviewed to determine that reportability requirements were fulfilled, immediate corrective action was accomplished, and corrective action to prevent recurrence had been accomplished in accordance with technical specifications.

Unit No. 1

LER No.	Title
1-80-60	Setpoint drift of 1B core spray discharge pressure transmitter 1-E21-PT-N001B. - GE Model No. 551
1-80-62	Control rod position indication reed switch problems.
1-80-64	1B1 battery amplifier control board battery charger output voltage problem.

Unit No. 2

2-80-39	Failure of paddle bushing on Bergen-Paterson snubber-Model No. HSSA-3, No. 2G41-12SS33.
2-80-42	Reactor low pressure switch J of 2-B21-PS-N021B setpoint drift.
2-80-57	Reactor low water level switch 2-B21-LIS-N031C-3 setpoint drift.

The corrective action indicated for each of the above events were determined to be adequate. No items of noncompliance or deviations were identified.

7. IE Bulletin Followup

For the IE Bulletins listed below the inspector verified that the written response was within the time period stated in the bulletin. that the written response included the information required to be reported, that the written response included adequate corrective action commitments based on information presented in the bulletin and the licensee's response, that licensee management forwarded copies of the written response to the appropriate onsite management representatives, that information discussed in the licensee's written response was accurate, and that corrective action taken by the licensee was as described in the written response.

IEB No.	Title
IEB-80-15	Possible loss of hotline with loss of offsite power (reference CP&L letter to NRC S/N 80-80126 of August 27, 1980)

IEB No.	Title
IEB-80-19	Failure of Mercury-wetted matrix relays in reactor protective systems of operating nuclear power plants designed by Combustion Engineering (reference CP&L letter to NRC S/N 80-1222 of August 19, 1980).
IEB-80-20	Failure of Westinghouse type W-2 Spring return to Neutral control switches. (reference CP&L letter S/N 80-1221 of August 19, 1980).

The action taken on these Bulletins has been determined to be adequate. No items of noncompliance or deviations were identified in the review of action taken by the licensee.

8. IE Circular Followup

For the IE Circulars listed below, the inspector verified that the Circular was received by the licensee management, that a review for applicability was performed, and that if the circular were applicable to the facility appropriate corrective actions were taken or were schedule to be taken.

IEC No.	Title
IEC-80-18	10 CFR 50.59, Safety evaluation for changes to radwaste treatment systems
IEC-80-19	Noncompliance with licensee requirements for Medical Licensees (Not aplicable).
IEC-80-20	Changes in safe-slab tank dimensions. (Not applicable).

The licensee action on the above circulars was determined to be adequate. No items of noncompliance or deviations were identified.

9. Acceptance Criteria

The inspector used one or more of the following sources of acceptance criteria for evaluating the above areas inspected.

- a - 10 CFR 50, Appendix "B"
- b - 10 CFR 20
- c - Technical Specifications
- d - ANSI N.45.2.3 (1971)
- e - ANSI N.18.7 (1972)
- f - Regulatory
- f - Regulatory Guides
- g - Site Security Plan
- h - Quality Assurance Program