



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

YANKEE ATOMIC ELECTRIC COMPANY

DOCKET NO. 50-29

YANKEE NUCLEAR POWER STATION (YANKEE-ROWE)

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29  
License No. DPR-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Yankee Atomic Electric Company (the licensee) dated August 18, 1975 and supplement dated January 29, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Part 50, Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. After weighing the environmental aspects involved, the issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

8011030 786

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Charles M. Trammell*  
A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: July 22, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 29

FACILITY OPERATING LICENSE NO. DPR-3

DOCKET NO. 50-29

Revise Appendix A as follows:

Remove page 19 and insert revised page 19

Add pages 19a and 19b and Table F.2-1

F. LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

F.1 ROD DROP TIME

LIMITING CONDITION FOR OPERATION

The individual control rod drop times shall be  $\leq 2.50$  seconds from loss of stationary gripper coil voltage to 6 inch coil entry with:

- a.  $T_{avg} \geq 511^{\circ}$  and;
- b. all reactor coolant pumps operating.

APPLICABILITY: Hot Standby.

ACTION:

- a. With the drop time of any rod determined to exceed the above limit, restore the drop time to within the above limit prior to proceeding to startup or power operation.
- b. With the rod drop times within limits but determined with 3 reactor coolant pumps operating, operation may proceed provided thermal power is restricted to  $\leq 75$  percent of rated thermal power.

SURVEILLANCE REQUIREMENTS

The rod drop time of control rods shall be demonstrated through measurement prior to reactor criticality:

- a. for all rods following each removal of the reactor vessel head;
- b. for specifically affected individual rods following any maintenance on or modification to the rod drive system which could affect the drop time of those specific rods, and;
- c. at least once per 18 months

F.2 SHOCK SUPPRESSORS (SNUBBERS)

LIMITING CONDITIONS

During all modes of operation except Cold Shutdown and Refueling, all snubbers listed in Table F.2-1 shall be operable except as described below.

- a. Reactor operation is only permitted for 72 hours after the time a snubber is determined to be inoperable unless the snubber is sooner made operable or replaced.
- b. If the requirements of "a" cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 36 hours.
- c. Snubbers may be added to safety related systems without prior License Amendment to Table F.2-1 provided that a revision to Table F.2-1 is included with a subsequent License Amendment request.
- d. If a snubber is determined to be inoperable while the reactor is in a cold shutdown or refueling mode, the snubber shall be made operable or replaced prior to exceeding the cold shutdown mode.

APPLICABILITY: All modes of operation except for Cold Shutdown and Refueling.

SURVEILLANCE REQUIREMENTS

The following surveillance requirements shall apply to all hydraulic snubbers listed in Table F.2-1.

- a. All hydraulic snubbers whose seal materials has been demonstrated by operating experience, lab testing or analysis to be compatible with the operating environment shall be visually inspected in accordance with the following schedule:

<u>Number of snubbers found inoperable during inspection or inspection interval</u>	<u>Next required inspection interval</u>
0	18 months + 25%
1	12 months + 25%
2	6 months + 25%
3, 4	124 days + 25%
5, 6, 7	62 days + 25%
≥ 8	31 days + 25%

The required inspection interval shall not be lengthened more than one step at a time.

The initial inspection shall be performed within six months from the date of issuance of these specifications. It shall be assumed that the facility has been on a six months, + 25% inspection interval.

- b. All hydraulic snubbers whose seal materials are other than ethylene propylene or other material that has been demonstrated to be compatible with the operating environment shall be visually inspected for operability every 31 days.
- c. The visual inspection to verify snubber operability shall include, but not necessarily limited to: inspection of the hydraulic fluid reservoir, fluid connections, and linkage connections to the piping and anchor.
- d. Once each refueling cycle, a representative sample of approximately 10% of the hydraulic snubbers shall be functionally tested for operability including verification of proper piston movement, lock-up and bleed. For each unit and subsequent unit tested that proves inoperable, an additional approximately 10% of the snubbers shall be functionally tested until no more failures are found or all snubbers have been tested. Snubbers of rated capacity greater than 50,000 lbs. need not be functionally tested.

Table F.2-1

SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS)

<u>Snubber No.</u>	<u>Protected System</u>	<u>Location</u>	<u>Accessible During Normal Operation</u>
HSS 19A HSS 19B HSS 20A HSS 20B	Pressurizer Relief Valve	Pressurizer Cubicle	Yes
HSS 21 HSS 22 HSS 23 HSS 24	Pressurizer Relief Valve Discharge Line	Pressurizer Cubicle	Yes