

YANKEE ATOMIC ELECTRIC COMPANY



ROWE, MASSACHUSETTS 01367

March 27, 1973



United States Atomic Energy Commission  
Washington, D. C. 20545

Attention: Directorate of Licensing

Reference: License No. DPR-3 (Docket 50-29)

Dear Sirs:

In accordance with Section D (1) (b) of the plant license we are reporting the following variance in the pressurizer code safety valve set pressures.

The pressurizer safety valves are provided to prevent system pressure from exceeding design limits. The two pressurizer code safety valves are Dresser 2" x 3" Model 1719 WA valves with a capacity of 92,000 pounds/hour/valve. The safety valve designations and set points are:

PR-SV-181	2485 psig
PR-SV-182	2560 psig

These safety valves are removed and tested on a test stand with nitrogen at refueling intervals for pressure settings and spring load data. Both have had a past history of conformance with the ASME Code requirements. However during the Core 9-10 refueling in March 1972, the "as found" set point pressures were out of tolerance.

PR-SV-181	Was 175 pounds above the code set point limit.
PR-SV-182	Was 174 pounds above the code set point limit.

The valves were disassembled and thoroughly inspected. No abnormal conditions were noted during this inspection. The valves were reset on the test stand to the correct set points.

These same safety valves were removed and retested in November 1972 under the direction of a Dresser Industries representative. Again a variance was noted in the "as found" pressure set points with both valves lifting above the required settings.

The Dresser representative disassembled and inspected both safety valves. The spindles were found to be bent between .012" - .014". Subsequent investigation by the service engineer concluded that one phase of the existing test procedure could have a detrimental effect on the valve spindles. In his opinion, measuring the safety valve spring loading during deflection could result in spindle distortion. This test was normally performed with the valve fully assembled, and warned of possible spring tension loss.

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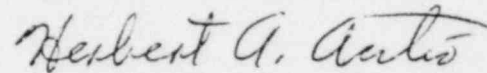
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The pressurizer safety valve test procedure was revised to eliminate this problem. Subsequent tests confirmed repeated lifts within code tolerances.

The pressurizer code safety valves will be removed and retested during the next major plant outage. This time interval to be no earlier than six months and no later than the Core 10-11 refueling outage. The valve vendor will be consulted before and during this next test.

A safety evaluation has been completed of the effects of the 175 psi overpressure on system component and transient conditions. The evaluation indicates that no system or transient safety problem would have resulted if the overpressure condition had occurred.

Very truly yours,



Herbert A. Autio  
Plant Superintendent

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cc: J. O'Reilly

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CLASS:	INPUT	NO CYS REC'D		DOCKET NO:			
U/PROP INFO		40		50-29			
DESCRIPTION:				ENCLOSURES:			
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PLANT NAMES: Yankee							

FOR ACTION/INFORMATION 3-31-73 AB

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POOR ORIGINAL

Yankee Atomic Electric Company

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cc: Mr. Donald G. Allen, President  
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