



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION III  
799 ROOSEVELT ROAD  
GLEN ELLYN, ILLINOIS 60137

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OCT 21 1980

Docket No. 50-373  
Docket No. 50-374

Commonwealth Edison Company  
ATTN: Mr. Cordell Reed  
Vice President  
Post Office Box 767  
Chicago, IL 60690

Gentlemen:

Thank you for your final report dated October 8, 1980, pursuant to 10 CFR 50.55(e) regarding small pipe hanger deficiency. We will complete our review of this matter during a future inspection.

Your cooperation with us is appreciated.

Sincerely,

*G. Fiorelli*  
for G. Fiorelli, Chief  
Reactor Construction and  
Engineering Support Branch

cc w/ltr dtd 10/8/80:  
Mr. J. S. Abel, Director  
of Nuclear Licensing  
Mr. L. J. Burke, Site  
Construction Superintendent  
Mr. T. E. Quaka, Quality  
Assurance Supervisor  
Mr. R. H. Holyoak, Station  
Superintendent  
Mr. B. B. Stephenson  
Project Manager  
Central Files  
Reproduction Unit NRC 20b  
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Assistant Attorney General

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**Commonwealth Edison**  
One First National Plaza Chicago Illinois  
Address Reply to Post Office Box 767  
Chicago, Illinois 60690

October 8, 1980

Mr. James G. Keppler, Director  
Directorate of Inspection and  
Enforcement - Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Subject: LaSalle County Station Unit 1  
Small Pipe Hanger Design Deficiency  
10 CFR 50.55(e) Final Report  
NRC Docket No. 50-373

Reference (a): J. S. Abel letter to J. G. Keppler dated  
September 26, 1980

Dear Mr. Keppler:

Commonwealth Edison notified Mr. Cordell Williams of your office on September 10, 1980, of safety related 2" & under pipe hanger deficiencies at LaSalle County Station Unit #1. Specifically, a recent NRC inspection at the site identified the following items associated with 2" and under pipe hanger design work:

- (1) Design work was being performed using guidelines in lieu of approved procedures.
- (2) Design review standards were not being maintained as controlled documents.
- (3) Site oriented training of personnel involved in design was not being documented.
- (4) Interface documents were not formally established to control the design review process.
- (5) QA audits had not been completed to cover the design activities at the site.

Due to these deficiencies, documented calculations for the support steel and load and location do not exist for a portion of the safety related 2" & under pipe hangers. Approximately 543 piping subsystems are involved in this problem. The existing review program has been more clearly documented to assure that the A-E reviews the installed hangers and performs calculations where

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Mr. James G. Keppler, Director  
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necessary to document the adequacy of the hangers. The items noted by the NRC have been corrected. The Commonwealth Edison QA Dept. has performed extensive audits in the area of design control and review. In addition, a surveillance program to monitor the continuing work has been instituted.

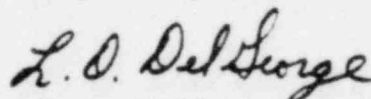
Although these problems reflect upon the adequacy of the Quality Assurance program applicable to the small bore pipe hangers, it is judged based on the follow-up audits performed and the findings uncovered and reported to the NRC, that the QA deficiencies to the extent they exist were limited to the field design interface associated with small bore pipe hangers. The corrective action discussed with the NRC (Region III) on August 29 and September 5, 1980 are expected to resolve these apparent deficiencies and will be the basis for closing this report under 10 CFR 50.55(e).

In this regard, the program for reporting progress on the corrective action to be taken was discussed in the meeting with your staff on September 5, 1980. It was subsequently agreed by Commonwealth Edison that periodic reports would be submitted to discuss the progress of the work. As mandated by your Staff, these reports will be initiated two weeks after receipt of the formal NRC inspection report. This inspection report has not been received as of the date of this letter.

Because the commitment to formal reporting of progress on corrective action has been documented in Item 3 of the Enclosure to Reference (a), no further reports under 10 CFR 50.55(e) are judged necessary. This conclusion is supported by the fact that a comprehensive review of the subject deficiencies has already been completed by your staff and the primary purpose of the future reports is to provide a completion status.

If there are any further questions in this regard, please direct them to this office.

Very truly yours,



L. O. DelGeorge  
Nuclear Licensing Administrator  
LaSalle County Station

LOD/rap  
cc: Director of Inspection and  
Enforcement, DC  
RIII Inspector LSCS