October 17, 1980

50-409

In reply, please refer to LAC-7185

Mr. James G. Keppler, Regional Director U. S. Nuclear Regulatory Commission Directorate of Regulatory Operations Region III 799 Roosevelt Road Glen Ellyn, IL 60137

SUBJECT: DAIRYLAND POWER COOPERATIVE LA CROSSE BOILING WATER REACTOR PROVISIONAL OPERATING LICENSE NO. DPR-45

SPECIAL REPORT 80-02

REFERENCE: (1) LACBWR Technical Specifications Section 4.2.2.22, Action g.

Dear Mr. Keppler:

In accordance with the provisions of Reference (1), a Special Report is submitted covering results of sampling and analysis actions together with additional operational information for five separate occasions when reactor thermal power changed by more than 15% of Rated Thermal Power within one hour while in Operational Condition 1 or 2.

The separate occasions are discussed in sections as follows:

SECTION 1----July 19, 1980 SECTION 2----August 8, 1980 SECTION 3----August 32, 1980 SECTION 4----September 5, 1980 SECTION 5----October 4, 1980

Each section provides a summary of operating data and radiological data for the time period prior to the thermal power change along with the results of the additional sampling required.

If there are any questions concerning this report, please contact us.

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Mr. James G. Keppler, Regional Director U. S. Nuclear Regulatory Commission

LAC-7185 October 17, 1980

Very truly yours,

Frank Linder, General Manager

FL: LSG: abs

#### ATTACHMENTS

CC: Director, Office of Inspection and Enforcement (30)
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Director, Office of Management, Information, and Program Control (3) U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Resident Inspectors

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Raffety Angle Towsley

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Krajewski Goodman

Files: Reading

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A10, F3, H1, T5N

#### INTRODUCTION

LACBWR Technical Specifications, Section 4.2.2.22, Action g, requires sampling and analysis for Iodine-131, -133, and -135, as well as gross beta and gamma activity between 2 and 6 hours following a thermal power change of greater than 15% of rated thermal power within one hour. This special report is required to contain the information determined by the analysis plus additional information regarding reactor power history, fuel burnup, cleanup flow history, offgas activity levels, and gross alpha activity levels.

#### SECTION 1

1B Forced Circulation Pump tripped at 2214 on July 19, 1980, due to low seal inject leakoff flow.

The reactor was operating at 85% of rated thermal power (37 MWe-Net) at the time of the 1B FCP trip. The pump trip caused reactor power to drop to 49% rated thermal power.

#### GROSS BY ACTIVITY AND IODINE ANALYSIS

Time of Sample: 0200, 7/20,80

Sample Results: Gross  $\beta\gamma$ ----1.07  $\mu$ Ci/gm I-131----5.19 x 10<sup>-3</sup>  $\mu$ C

I-131-----5.19 x  $10^{-3}$   $_{\mu}\text{Ci/gm}$  I-133-----2.29 x  $10^{-2}$   $_{\mu}\text{Ci/gm}$  I-135-----1.05 x  $10^{-2}$   $_{\mu}\text{Ci/gm}$ 

#### ADDITIONAL INFORMATION

(1) Reactor Thermal Power Level starting 48 hours prior to thermal power change:

0200, 7/17/80 - 2214, 7/19/80-----85%

- (2) The approximate fuel burnup of assemblies in a symmetrical core region is depicted in Figure 1.
- (3) Cleanup Flow History starting 48 hours prior to the thermal power change:

## Primary Purification Flow Rate

2200, 7/17/80 - 2214, 7/19/80-----41 GPM

## Primary Purification Decontamination Factors (BY)

0025, 7/17/80-----114:1

0226, 7/21/80----196:1

(4) Offgas Activity Levels for 48 Hours Prior to Thermal Power Change:

1600-2400,	7/17/80261	Ci/day
0000-0800,	7 12 2 12 2	Ci/day
0800-1600,		Ci/day
1600-2400,	7/18/80265	Ci/day
0000-0800,	7/19/80269	Ci/day
0800-1600,	7 /10 /00	Ci/day

(5) Gross Alpha Activity Level Starting With the Last Sample Taken Prior to Thermal Power Change:

0023,	7/17/80 2.14 x 10-8 uCi/qm	
2214,	7/19/80Thermal Power Change	
0225,	7/21/803.82 x 10-8 µCi/gm	

#### SECTION. 2

A reactor scram occurred at 0843 on August 8, 1980, when the Forced Circulation Pumps tripped due to low seal inject flow.

The reactor was operating at 85% of rated thermal power (37 MWe-Net) at the time of the scram.

## GROSS 87 ACTIVITY AND IODINE ANALYSIS

Time of Sample: 1307, 8/8/80

## ADDITIONAL INFORMATION

(1) Reactr Thermal Power Level starting 48 hours prior to thermal power change:

0800, 8/6/80 - 0843, 8/7/80-----85%

- (2) The approximate fuel purnup of assemblies in a symmetrical core region is depicted in Figure 2.
- (3) Cleanup Flow History starting 48 hours prior to the thermal power change:

## Primary Purification Flow Rate

0800, 8/6/80 - 0843, 8/8/80-----41 GPM

## Primary Purification Decontamination Factors (BY)

0102, 8/7/80----134:1 0103, 8/14/80----75:1

(4) Offgas Activity Levels for 48 Hours Prior to Thermal Power Change:

0800-1600,	8/6/80268	Ci/day
1600-2400,		Ci/day
0000-0800,	8/7/80293	Ci/day
0800-1600,		Ci/day
1600-2400,	8/7/80280	Ci/day
0000-0800,		Ci/day

(5) Gross Alpha Activity Level Starting With the Last Sample Taken Prior to Thermal Power Change:

0104,	8/7/806.1 x 10 <sup>-7</sup> µCi/qm
0843,	8/8/80Thermal Power Change
1307,	8/8/806.96 x 10 <sup>-7</sup> µCi/gm

#### SECTION 3

A reactor scram occurred at 1430 on August 22, 1980, when the 1A Reactor Feed Pump (RFP) tripped due to water dripping onto and shorting out a portion of the RFP controls.

The reactor had achieved 55% rated thermal power during a slow escalation when the scram occurred.

#### GROSS BY ACTIVITY AND IODINE ANALYSIS

Time of Sample: 1648, 8/22/80

#### ADDITIONAL INFORMATION

(1) Reactor Thermal Power Level starting 48 hours prior to thermal power change:

```
1400-2400, 8/20/80-----45% + 49%
0000-2400, 8/21/80-----49% + 51%
0000-1430, 8/22/80----51% + 55%
```

(2) The approximate fuel burnup of assemblies in a symmetrical core region is depicted in Figure 2.

(3) Cleanup Flow History starting 48 hours prior to the thermal power change:

## Primary Purification Flow Rate

1400, 8/20/80 - 1430, 8/22/80-----41 GPM

## Primary Purification Decontamination Factor (84)

0041, 8/21/80-----68:1 0045, 8/25/80----181:1

# (4) Offgas Activity Levels for 48 Hours Prior to Thermal Power Change:

0800-1600,	8/20/80137	Ci/day
1600-2400,		Ci/day
,0090-0000,		Ci/day
0800-1600,		Ci/day
1600-2400,		Ci/day
0000-0800,	8/22/80165	Ci/day

## (5) Gross Alpha Activity Level Starting With the Last Sample Taken Prior to Thermal Power Change:

0033,	8/21/802.12 x 10-7 µCi/gm
1430,	8/22/80Thermal Power Change
1648,	8/22/808.54 x 10 <sup>-7</sup> µCi/gm

## SECTION. 4

A partial scram occurred at 0700 on September 5, 1980, when water leakage from the seal inject supply line connection to Control Rod Drive Mechanism (CRDM) No. 1 sprayed onto the scram solenoid area of CRDM No. 3 causing an electrical short.

The reactor had achieved 31% rated thermal power during a slow escalation prior to the scram.

## GROSS, BY. ACTIVITY AND IODINE ANALYSIS

Time of Sample: 1115, 9/5/80

Sample Results: Gross gy------9.34 x  $10^{-2}$   $_{\mu}$ Ci/gm I-131------3.737 x  $10^{-4}$   $_{\mu}$ Ci/gm I-135------5.774 x  $10^{-3}$   $_{\mu}$ Ci/gm

#### ADDITIONAL INFORMATION

(1) Reactor Thermal Power Level starting 48 hours prior to thermal power change:

- (2) The approximate fuel burnup of assemblies in a symmetrical core region is depicted in Figure 2.
- (3) Cleanup Flow History starting 48 hours prior to the thermal power change:

## Primary Purification Flow Rate

## Primary Purification Decontamination Factor (BY)

```
0602, 9/4/80----35.7:1
0231, 9/8/80----36.7:1
```

# (4) Offgas Activity Levels for 48 Hours Prior to Thermal Power Change:

## (5) Gross Alpha Activity Level Starting With the Last Sample Taken Prior to Thermal Power Change:

#### SECTION 5

A reactor scram occurred at 0732 on October 4 when the 13 Reactor Feed Pump controller failed, increasing reactor water level to the high water level setpoint.

The reactor was operating at 85% rated thermal power at the time of the scram.

## GROSS. &Y. ACTIVITY AND IODINE ANALYSIS

Time of Sample: 0942, 10/4/80

Sample Results: Gross 87-----.250 µCi/gm

I-131-----2.19 x 10<sup>-3</sup> LCi/gm I-133-----1.78 x 10<sup>-2</sup> LCi/gm I-135-----2.61 x 10<sup>-2</sup> LCi/gm

#### ADDITIONAL INFORMATION

(1) Reactor Thermal Power Level starting 48 hours prior to thermal power change:

0700, 10/1/80 - 0732, 10/4/80----85%

- (2) The approximate fuel burnup of assemblies in a symmetrical core region is depicted in Figure 3.
- (3) Cleanup Flow History starting 48 hours prior to the thermal power change:

## Primary Purification Flow Rate

## Primary Purification Decontamination Factor (By)

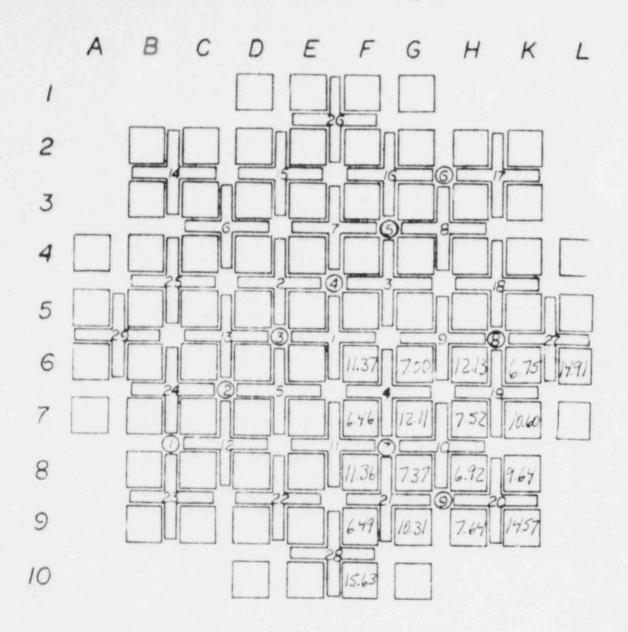
0116, 10/2/80-----62:1 0109, 10/6/80-----4.28:1 0036, 10/9/80-----38.9:1

## (4) Offgas Activity Levels for 48 Hours Prior to Thermal Power Change:

# (5) Gross-Alpha Activity Level Starting With the Last Sample Taken Prior to Thermal Power Change:

0115, 10/2/80------1.22 x  $10^{-7}$   $\mu$ Ci/gm 0732, 10/4/80------Thermal Power Change 1257, 10/5/80-----7.04 x  $10^{-7}$   $\mu$ Ci/gm

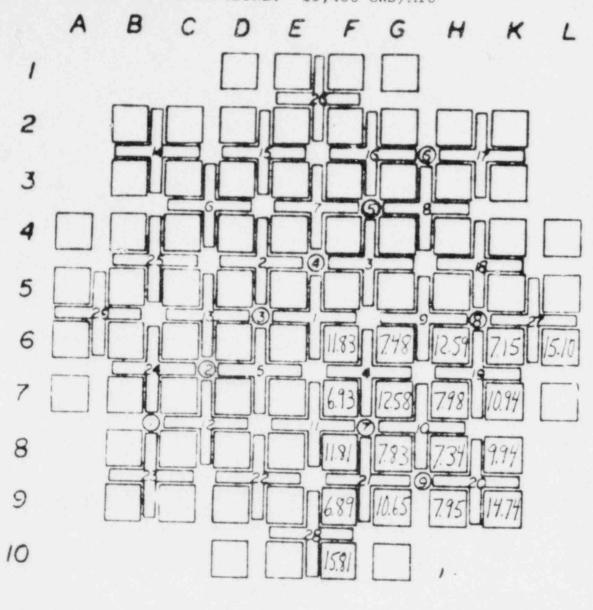
QUARTER CORE FUEL EXPOSURE ESTIMATION (GWD/MTU) AN INDICATION OF REGIONAL EXPOSURE AS OF JULY 19, 1980 THE AVERAGE EXPOSURE: 10.117 GWD/MTU



PLANT NORTH

IN CORE FLUX MONITORS

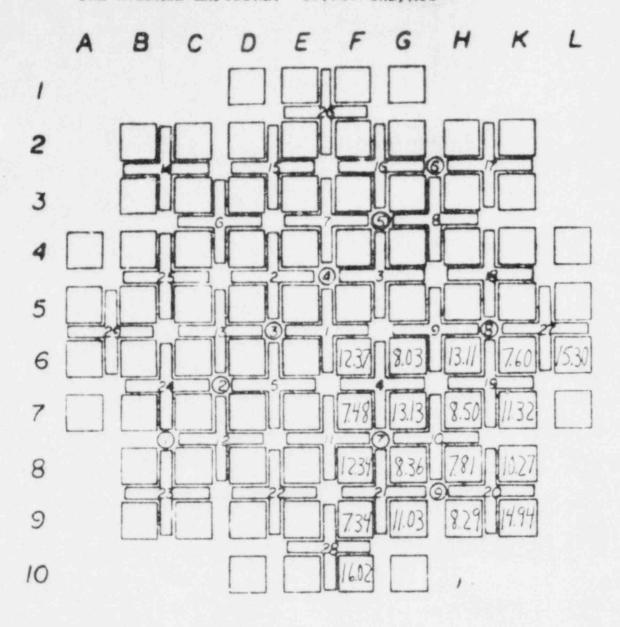
FUEL EXPOSURE ESTIMATION (GWD/MTU)
AN INDICATION OF EXPOSURE ON AUGUST 31, 1980
THE AVERAGE EXPOSURE: 10,486 GWD/MTU



PLANT NORTH

IN CORE FLUX MONITORS (

FUEL EXPOSURE ESTIMATION (GWD/MTU)
AN INDICATION OF EXPOSURE ON SEPTEMBER 30, 1980
THE AVERAGE EXPOSURE: 10,909 GWD/MTU



PLANT NORTH

IN CORE FLUX MONITORS (