



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
PHILADELPHIA ELECTRIC COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNITS NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 3  
License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The two applications for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated August 22, 1980 comply with the standards and requirements of the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the applications, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. This issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 3, are hereby incorporated in the license. Public Service Electric and Gas Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*B. C. Buckley for:*  
Frank J. Miraglia, Acting Chief  
Licensing Branch No. 3  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: OCT 10 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 3

FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

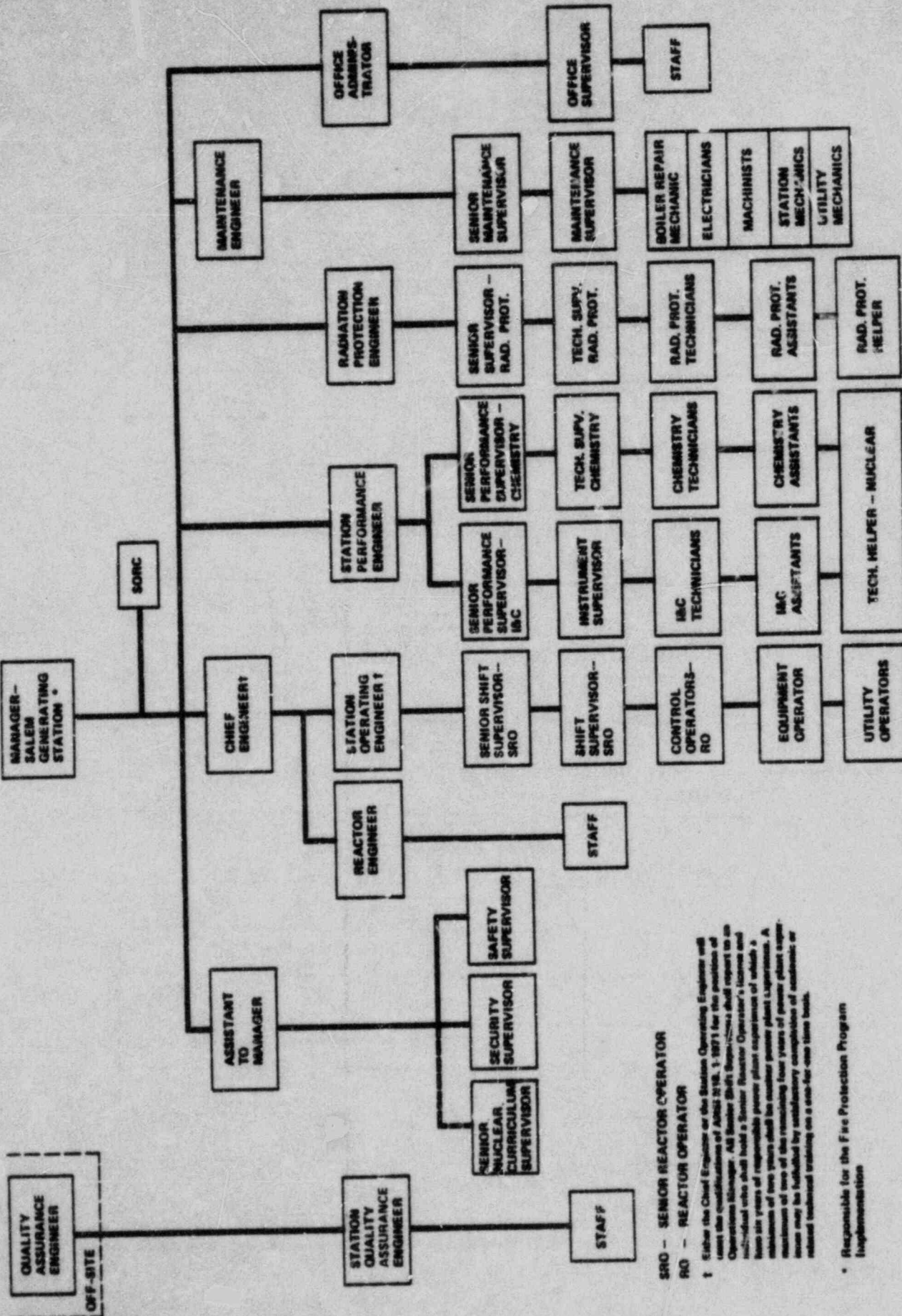
Revise Appendix A as follows:

Remove Pages

6-3  
6-6  
6-18  
6-19  
8-2

Insert Pages

6-3  
6-6  
6-18  
6-19  
8-2



SALEM-UNIT 2

66

SRO - SENIOR REACTOR OPERATOR  
 RO - REACTOR OPERATOR

† Either the Chief Engineer or the Station Operating Engineer will meet the qualifications of ANSI N18.5-1971 for the position of Operations Manager. All Senior Shift Supervisors shall report to an individual who shall hold a Senior Reactor Operator's license and have six years of responsible power plant experience of which a minimum of two years shall be nuclear power plant experience. A minimum of two of the remaining four years of power plant experience may be fulfilled by satisfactory completion of academic or related technical training on a one-for-one time basis.

\* Responsible for the Fire Protection Program Implementation

FIGURE 6.2.2 FACILITY ORGANIZATION - SALEM GENERATING STATION

**TABLE 6.2-1**

**MINIMUM SHIFT CREW COMPOSITION<sup>#</sup>**

**Condition of Unit 2 - No Fuel in Unit 1**

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL	2	1*
OL	2	1
Non-Licensed	2	1

**Condition of Unit 2 - Unit 1 in MODES 1, 2, 3 or 4**

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL	4	3***
Non-Licensed	4	3

**Condition of Unit 2 - Unit 1 in MODES 5 or 6**

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL	3***	2
Non-Licensed	3	3

TABLE 6.2-1 (Continued)

\*Does not include the licensed Senior Reactor Operator supervising CORE ALTERATIONS.

\*\*Each unit will be supervised by a shift supervisor who is a licensed SRO on that unit; or by a single individual if he is licensed on both units. The second senior operator licensed for each unit must be stationed in the control room area at all times when the unit is in operating modes 1 through 4; and could also be a single individual if he is licensed on Units 1 and 2.

\*\*\*A reactor operator licensed for each unit must be at the controls of that unit at all times when fuel is in the reactor. Also, a relief reactor operator licensed for each unit must be available on-shift. This could be a single individual if he is licensed for both units.

#Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

## ADMINISTRATIVE CONTROLS

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Radiation Protection Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

### 6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be coordinated by the Assistant to Manager and maintained under the direction of the Training Engineer and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 STATION OPERATIONS REVIEW COMMITTEE (SORC)

##### FUNCTION

6.5.1.1 The Station Operations Review Committee shall function to advise the Station Manager on all matters related to nuclear safety.

##### COMPOSITION

6.5.1.2 The Station Operations Review Committee shall be composed of the:

Chairman:	Chief Engineer
Vice Chairman:	Assistant to Manager
Member:	Station Operating Engineer
Member:	Station Performance Engineer
Member:	Reactor Engineer
Member:	Senior Shift Supervisor
Member:	Senior Performance Supervisor - I&C
Member:	Senior Performance Supervisor - Chemistry
Member:	Radiation Protection Engineer
Member:	Senior Maintenance Supervisor
Member:	Maintenance Engineer

##### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

##### MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

## ADMINISTRATIVE CONTROLS

### 6.10 RECORD RETENTION

In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.10 . The following records shall be retained for at least five years:

- a. Records and logs of unit operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. All REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures required by Specification 6.8.1.
- g. Records of radioactive shipments.
- h. Records of sealed source and fission detector leak tests and results.
- i. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Unit Operating License:

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.



## ADMINISTRATIVE CONTROLS

- e. Records of transient or operational cycles for those facilities identified in Table 5.7-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or reviews of tests and experiments pursuant to 10 CFR 50.55.
- k. Records of meetings of the SORC and the NRB.

### 6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

### 6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Exposure Permit<sup>2</sup>. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of the same.

<sup>2</sup>Radiation Protection personnel or personnel escorted by Radiation Protection personnel shall be exempt from the REP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

## ADMINISTRATIVE CONTROLS

- c. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Senior Supervisor - Radiation Protection in the Radiation Exposure Permit.

6.12.2 The requirements of 6.12.1, above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Senior Shift Supervisor on duty and/or the Senior Supervisor - Radiation Protection.

8.0 SELECTED MATTERS TO BE COMPLETED AFTER  
COMPLETION OF THE LOW POWER TEST PROGRAM

- 8.1 The licensees shall perform secondary flow stability tests prior to achieving 100 percent power. The licensees shall submit and obtain NRC approval of these tests. The licensees shall provide at least 24 hours notification to IE prior to conducting such approved tests so that these tests may be witnessed by the NRC.
- 8.2 The licensees shall not operate the reactor at power levels above P-7 (as defined in Table 3.3-1 of Specification 3.3.1.1 of Appendix A to this license) with fewer than four reactor coolant loops in operation until safety analyses for fewer than four loop operation have been submitted and approval for fewer than four loop operation at power levels above P-7 has been granted by the Commission by Amendment of this license.
- 8.3 Prior to startup following the first regularly scheduled refueling outage, the licensees shall implement the fire protection modifications as described in the NRC Safety Evaluation Report, "Salem Nuclear Generating Station, Unit Nos. 1 and 2 Fire Protection Safety Evaluation Report" (Appendix E to Supplement No. 4 to the Salem Safety Evaluation Report, NUREG-0517).
- 8.4 Prior to the startup following the first regularly scheduled refueling outage, the licensees shall specify the details of the inspection program for guide thimble tube wall wear.
- 8.5 The licensees shall take the following remedial actions, or alternative actions acceptable to the NRC staff, with regards to the environmental qualification requirements appropriate to Class 1E equipment:
- (1) The wide-range resistance temperature detectors for the reactor coolant system shall be qualified for the radiation exposure for 40-year plant life and appropriate exposure contribution due to

design basis accidents. Pending completion of such qualification and acceptance by the NRC, each of these detectors shall be replaced at each regularly scheduled refueling outage.

- (2) Prior to completion of the second scheduled refueling outage, pressure transmitters and differential pressure transmitters from Barton Lot I shall have been replaced by suitably qualified devices.
- (3) Prior to completion of the first refueling outage, the Scotchcast No. 9 resin seals, used at the electrical connection interface on the NAMCO limit switches, will be replaced with Conax Electric Conduction Seal Assemblies.

8.6 Deleted.

8.7 Prior to exceeding five percent power, the licensees shall plug the Row 1 tubes in the steam generators.

8.8 At each of the first three regularly scheduled refueling outages, the licensees shall perform an ultrasonic inspection of the channelhead in the No. 21 steam generator in a selected area.

8.9 Prior to exceeding five percent power, the licensees shall install a reset alarm for the containment purge and pressure-vacuum relief valve reset circuitry.

8.10 Prior to startup following the first regularly scheduled refueling outage, the licensees shall submit the confirmatory results of the containment sump model test program, along with a description of any sump modifications resulting from the tests.