

UNITED STATES NUCLEAR REGULATORY COMMISSION REGION III 799 ROOSEVELT ROAD GLEN ELLYN, ILLINOIS 60137

SEP 2 0 1080

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Docket No. 50-151 Docket No. 50-356

University of Illinois ATTN: Dr. George ". Miley Chairman Nuclear Engineering Executive Committee 216 Nuclear Engineering Laboratory Urbana, IL 61801

Gentlemen:

Thank you for your letter dated September 9, 1980, informing us of the steps you have taken to correct the noncompliance identified in our letter dated September 2, 1980. We will examine your corrective action during a future inspection.

In your response you stated that an error was noted under Item 15 of our report. We acknowledge this error and it has been corrected. Enclosed is a corrected copy of Page 7 of our Reports No. 50-151/80-02 and 50-356/80-02 which incorporates the change.

Your cooperation with us is appreciated.

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Sincerely,

aB2 amo

A. B. Davis, Chief Fuel Facility and Materials Safety Branch

cc w/ltr dtd 9/9/80: Central Files Reproduction Unit NRC 20b PDR Local PDR NSIC TIC waste has been generated since August 1979. A review of the licensee's response to IE Bulletin 79-19 was performed during an inspection of the University broad license in March 1980.

14. Transfer of Radioactive Material

The licensee has not submitted a quality assurance program (10 CFR 71.51), because they have not shipped fuel and are not planning to ship fuel in the near future. The inspector informed the licensee that before any shipment of spent fuel they must submit a QA plan in accordance with 10 CFR 71.51.

The inspector reviewed the licensee's records of transfer of licensed material from September 1979 to date. Material is transferred to both on and off campus users. During the first six months of 1980 several shipments of irradiated Type A quantities of normal form rad'oactive materials were transferred to Argonne National Laboratory (ANL) in an ANL shipping container. Although the licensee maintains shipping papers on each shipment they do not maintain a complete certification and safety analyses demonstrating that the container meets specification 7A. This is in noncompliance with 49 CFR 173.395(a)(1).

No shipment of spent fuel reactor fuel has been made since the last inspection in August 1979.

15. Independent Measurements

Gamma and neutron surveys were made by the inspector and reactor health physicist in the reactor facility on August 20, 1980, while the reactor was operating at 500 kilowatts. The highest radiation fields found were 8 mrems/hour neutron and 5 mR/hr gamma at the face of the north beam port. All other areas were less than 3 mrems/hour, neutron and gamma combined.

16. Exit Interview

The inspector met with licensee representatives (denoted in Paragraph 1) at the conclusion of the inspection on August 20, 1980.

The following matters were discussed.

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- a. The purpose and scope of the inspection.
- b. The item of noncompliance (Paragraph 14).

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