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JOCKET HOS. (2-10)50-237/50-249 and 50-254/60-265

AUG 3 1977

Consonwealth Edison Company
ATTN: Mr. R. L. Bolger
Assistant Vice President
Post Office Box 767
Chicago, Illinois 60690

Gentleren:

RE: DRESDEN NUCLEAR POWER STATION UNIT NOS. 1, 2 AND 3 UUAD-CITIES NUCLEAR POWER STATION, UNIT NOS. 1 AND 2

In the past several years, a significant number of relief valves and safety-relief valves were found to be inoperable at BWR reactor facilities. These valves were installed in the Reactor Coolant System and/or Automatic Depressurization System. Several programs have been developed to reduce the incidence of these valve failures; however, additional failures continue to occur.

Consequently, we have concluded that changes to the Surveillance Requirements and Limiting Conditions for Operations for all BUR's are needed to provide additional assurance of relief valve and safety-relief valve operability and reliability. Therefore, we request that you modify your surveillance testing program through the adoption of the program contained in the model technical specifications we have prepared. The elements of this program include:

1. Each remotely operated relief valve and safety-relief valve in the Reactor Coolant System and Automatic Depressurization System will be tested on a variable frequency schedule related to demonstrated reliability and operability. The testing interval is based on the number of valve failures during the required test interval. Facilities with reliable valves will progress to a longer test interval while those with valve failures will progress to a sporter test interval. This concept should result in the maintenance of a more uniform level of reliability for this equipment than previously obtained.

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- The increased surveillance program will become effective on lurch i, 1979. No increase in valve testing is required before that date. The initial testing interval of the increased surveillance program will be based on the number of remotely operated relief valves and safety-relief valves found inoperable in the previous 18 months (September 1, 1977 to Harch 1, 1979). This lead time will permit the resolution of the Mark I Safety-Relief Valve Loads and Structural Capability generic concern. Additionally, this lead time is sufficient to penuit the development and implementation of improved safety and safety-relief valve maintenance procedures and other corrective actions prior to implementing the test program.
- 3. The relief and/or safety-relief valve line restraints in the torus will be examined prior to initiating the test program and at least once each fuel cycle (i.e., each 18 months) to verify continued structural integrity.

We request that you submit within 30 days from your receipt of this letter, an application for amendment to your license that will change your technical specifications to be in conformance with the requirements of the enclosed model technical specifications and associated bases. In the event you should desire further discussion of this matter, please contact us.

> Sincerely, Original signed by

Don K. Davis, Acting Chief Operating Reactors Branch #2 Division of Operating Reactors

Enclosure: Lodel Technical Specifications

cc a/enclosure: See next page

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