

Maine Yankee

RELIABLE ELECTRICITY FOR MAINE SINCE 1972

EDISON DRIVE • AUGUSTA, MAINE 04336 • (207) 622-4868

December 1, 1989

MN-89-146

GDW-89-372

UNITED STATES NUCLEAR REGULATORY COMMISSION

Attention: Document Control Desk

Washington, DC 20555

References: (a) License NO. DPR-36 (Docket No. 50-309)
(b) USNRC Letter dated October 19, 1989, Request for Information Concerning Status of Implementation of Unresolved Safety Issue (USI) Requirements (Generic Letter 89-21)

Subject: Response to Generic Letter 89-21, Implementation of Unresolved Safety Issue (USI) Requirements

Gentlemen:

Maine Yankee has reviewed the status of implementation of Unresolved Safety Issues, as requested and identified in Reference (b). Attached is an annotated copy of Enclosure (1) of Reference (b). We have marked the attachment with our judgement of status based on a review of our records for each item as applicable to Maine Yankee. This status report is not intended to make any commitments for Maine Yankee or modify any previous commitments. Also, please note the attachment provides expected dates of implementation for several pending items. Maine Yankee considers these dates tentative based on our current plans.

Should you have any questions on the attached, please contact me.

Very truly yours,

SE Nichols

for G. D. Whittier, Manager
Nuclear Engineering and Licensing

RCC:WSD

Attachment (Annotated Enclosure (1) of Generic Letter 89-21)

c: Mr. William T. Russell
Mr. Eric J. Leeds
Mr. Cornelius F. Holden

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ATTACHMENT

Annotated copy of Enclosure (1) of NRC Generic Letter 89-21

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ENCLOSURE 1UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUPEG-0737 Item I.A.2.3 SRP revisions	All	C/2-83	Attached
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	C/1980	Attached
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	NA	
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	C/5-88	Attached
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	NA	
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	NA	

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- * C - CHANGES WERE MADE AS A RESULT OF THIS USI AND NO OTHER CHANGES ARE PLANNED.
 NC - NO CHANGES WERE MADE AS A RESULT OF THIS USI.
 NA - NOT APPLICABLE
 I - CHANGES ARE PLANNED AS A RESULT OF THIS USI.
 E - USI UNDER EVALUATION.

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	NA	
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	NA	
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	I/92	Attached
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	NA	
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	All	NC	Attached
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWP	NA	Attached
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/77 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18 (No requirements)	All	I	Attached
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	C/6-84	Attached

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOP Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	C/3-81	Attached
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLS After 01/79.	NA	
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C/1-85	Attached
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SPP 6.2.1.1.C	BWR	NA	
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	NA	Attached
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	NA	

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.82 (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	All	C/12-82	Attached
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	All	C/6-89	Attached
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	All	I/12-91	Attached
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	All	C/12-88	Attached
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG- 1218 GL 89-19	All	E/3-90	Attached
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	All, except PWRs with large dry containments	NA	
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	NC	Attached

Maine Yankee Remarks to
Enclosure (1) of NRC Generic Letter 89-21

USI REMARKS (Correspondence unless otherwise indicated)

- A-1 Feedwater System:
NRC-MY, 04/05/83 - Inspection Report No. 50-309/83-01
NRC-MY, 03/18/83 - Inspection Report No. 50-309/83-03
MY-NRC, 02/18/83 (MN-83-31) - LER 83-002/01T-1, Main Feedwater System
MY-NRC, 02/17/83 (MN-83-29)
- A-2 CEOG-NRC, 11/20/87 (CEOG-87-662) - "Applicability of Leak-Before-Break
to the Primary Coolant System Piping of CE-Designed PWRs"
MY-NRC, 09/26/84 (MN-84-152) - Information to Support SER
NRC-MY, 05/31/84
MY-NRC, 07/01/80 (WMY 80-104)
MY-NRC, 05/14/80 (WMY 80-75)
- A-4 MY-NRC, 06/23/89 (MN-89-85) - Modify Commitment of MN-85-135
MY-NRC, 07/12/85 (MN-85-135) - Response to G.L. 85-02
- A-9 MY-NRC Due
NRC-MY, 08/03/89 - RAI
MY-NRC, 05/18/89 (MN-89-63)
- A-11 MY-NRC, 12/06/88 (MN-88-118)
NRC-MY, 11/20/86 - SER
WCAP-9875
- A-12 NA. Per Generic Letter 89-21: "Only applied to plants with a new
construction permit issued after October 1983."
- A-17 Remaining action is to complete plant specific PRA
MY-NRC, 10/31/89 (MN-89-126) - Response to G.L. 88-20, PRA Status
NRC-MY, 04/27/81 - Inspection Report No. 50-309/81-06, for TMI Task
Action Plan Item I.C.5
- A-24 NRC-MY, 10/23/89 - Inspection Report No. 50-309/89-16
NRC-MY, 11/19/87 - Inspection Report No. 50-309/87-16
MY-NRC, 01/25/85 (MN-85-17) - Certification of Compliance to 10 CFR 50.49
MY-NRC, 01/03/85 (MN-85-01) - Update
NRC-MY, 12/13/84 - SER
MY-NRC, 05/31/84 (MN-84-95) - Response to 04/08/83 SER
MY-NRC, 05/20/83 (MN-83-110) - 10 CFR 50.49 Response
NRC-MY, 04/08/83 - SER
MY-NRC, 09/02/81 (FMY 81-133) - Response
NRC-MY, 06/01/81 - SER and Order

Maine Yankee Remarks to
Enclosure (1) of NRC Generic Letter 89-21
(continued)

- USI REMARKS (Correspondence unless otherwise indicated)
- A-26 MY-NRC, 12/06/88 (MN-88-118) - Update for 10 CFR 50.61
MY-NRC, 12/02/88 (MN-88-116) - Proposed Change No. 145 to Technical Specifications in response to NRC Generic Letter 88-11
NRC-MY, 04/06/88 - Inspection Report No. 50-309/88-03, Section 11, Closeout of SIHS item A-26
NRC-MY, 02/27/81 - Technical Specification Amendment No. 53, Incorporated Overpressure Mitigation System (OMS)
MY-NRC, 10/16/79 (WMY 79-117) - Proposed Change No. 72 to Technical Specifications
NRC-MY, 09/11/79 - SER on OMS
MY-NRC, 02/24/78 (WMY 78-18) - OMS Technical Specifications, for information
- A-36 Phase I:
- NRC-MY, 03/21/86 - Inspection Report No. 50-309/86-01, Section 3.a
NRC-MY, 12/30/83 - SER
MY-NRC, 10/06/83 (MN-83-221)
MY-NRC, 05/27/83 (MN-83-115)
NRC-MY, 05/24/83 - R.A.I.
MY-NRC, 05/10/83 (MN-83-94)
MY-NRC, 08/27/82 (MN-82-169)
MY-NRC, 09/18/81 (FMY 81-141)
- Phase II:
- NRC-MY, 06/28/85 - Generic Letter 85-11 (letter stated that Phase II is considered completed.)
MY-NRC, 03/19/84 (MN-84-22)
- Maine Yankee Design Change EDCR 85-32 (OAS 7-5-7) - Single failure proof rewiring of AC solenoid brakes on reactor containment, turbine hall, and fuel building/yard cranes, in response to NRC Letter 12/19/83 (Generic Letter 83-42).
- A-40 See A-46
- A-43 Maine Yankee Design Change EDCR 82-30
- A-44 MY-NRC, 04/14/89 (MN-89-60)
- A-45 MY-NRC, 10/31/89 (MN-89-126) - Response to Generic Letter 88-20, PRA Status

Maine Yankee Remarks to
Enclosure (1) of NRC Generic Letter 89-21
(continued)

- A-46 NRC-MY, 08/31/87 - Audit of Plant Changes ... Seismic Upgrades
MY-NRC, 06/30/87 (MN-87-67) - Resolution of USI A-46
NRC-MY, 03/26/87 - SER - Seismic Margin Program
MY-NRC, 02/26/87 (MN-87-22)
MY-NRC, 03/24/86 (MN-86-45)
MY-NRC, 03/14/86 (MN-86-4i)
MY-NRC, 10/18/85 (MN-85-181)
NRC-MY, 10/29/84
- A-47 Response to NRC Generic Letter 89-19, dated 09/20/89, is due 03/19/90
- A-49 MY-NRC, 12/06/88 (MN-88-118) - Update per SER Request
NRC-MY, 11/20/86 - SER - 10 CFR 50.61
MY-NRC, 01/21/86 (MN-86-14) - Response to Requirement of 10 CFR 50.61
 (Pressurized Thermal Shock Rule)