



SACRAMENTO MUNICIPAL UTILITY DISTRICT □ 6201 S Street, P.O. Box 15830, Sacramento CA 95852-1830, (916) 452-3211  
AN ELECTRIC SYSTEM SERVING THE HEART OF CALIFORNIA

AGM/NUC 89-256

November 30, 1989

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

Docket No. 50-312  
Rancho Seco Nuclear Generating Station  
License No. DPR-54  
**REQUEST FOR INFORMATION CONCERNING STATUS OF IMPLEMENTATION OF UNRESOLVED  
SAFETY ISSUE (USI) REQUIREMENTS (GENERIC LETTER 89-21)**

Attention: George Knighton

Generic Letter 89-21 requested our review and reporting of the status of implementation of Unresolved Safety Issues (USI) for which the final technical resolution has been achieved.

The attached markup of Enclosure 1 of the Generic Letter indicates the status of each USI. Those items which are incomplete or under evaluation may be postponed indefinitely due to the status of Rancho Seco.

Members of your staff with questions requiring additional information or clarification may contact Steve Crunk at (209) 333-2935, extension 4913.

Sincerely,

Dan R. Keuter  
Assistant General Manager  
Nuclear

Attachment

cc w/atch: J. B. Martin, NRC, Walnut Creek  
A. D'Angelo, NRC, Rancho Seco  
G. Kalman, NRC, Rockville

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UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	All	C / 12-80	Mattimoe to Eisenhut letter dated 12/15/80 states Item I.A.2.3 is complete.
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	C / 2-86	SER of BWOOG Reports (2/18/86) stated acceptability of "leak-before-break" approach.
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	N/A	
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	N/A	
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	C / 6-85	Program description submitted via 6/11/85 Rodriguez to Thompson letter.
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	N/A	

\* C - COMPLETE  
 NC - NO CHANGES NECESSARY  
 NA - NOT APPLICABLE  
 I - INCOMPLETE  
 E - EVALUATING ACTIONS REQUIRED

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A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	N/A	
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	N/A	
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	I	Hardware changes remain - May be postponed indefinitely due to current plant status
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhuber dated 11/13/80 GL 81-11	BWP	N/A	
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	All	C / 4-86	D. Crutchfield to J. H. Taylor SER dated 4/30/86
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWP	N/A	Applicable to PWR w/CP later than 10/83
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/77 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18 (No requirements)	All	E	May be postponed indefinitely due to current plant status.
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	C / 11-85	Compliance achieved by 10 CFR 50.49 due date of 11/30/85

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A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOP Letters to Licensees 8/76 NUREG-G224 NUREG-0371 SRP 5.2 GL 88-11	PWR	I	Detailed review of LTOP setpoints remains. Postponed indefinitely due to current plant status.
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLs After 01/79.	N/A	
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C / 6-85	GL 85-11 (6/28/85) closed requirements.
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SPP 6.2.1.1.C	BWR	N/A	
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	I	SQUG Monitoring Issue and Necessary Changes
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	N/A	

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A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.87 (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	All	C / 3-89	Design review criteria upgraded 3/22/89
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	All	I	Hardware, procedure, training changes required. Postponed indefinitely due to current plant status.
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	All	N/C	IPE Subsumed issue and may no longer be applicable due to current plant status
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	All	I	Requirements and Approach under development
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG-1218 GL 89-19	All	E	Evaluation may be postponed indefinitely due to current plant status
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	All, except PWRs with large dry containments	N/A	
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	C / 1-86	Initial PTS Submittal of Jan. 1986 was first closure of this ongoing issue. GL 88-11 response required additional action