## ENCLOSURE 1

## NOTICE OF VIOLATION

Alabama Power Company Farley Docket Nos. 50-348, 50-364 License Nos. NPF-2, NPF-8

During the Nuclear Regulatory Commission (NRC) inspection conducted on October 2-6, 1989 and October 16-20, 1989, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (1989), the violations are listed below:

- A. Technical Specification 4.0.5 requires inservice testing of ASME Code Classes 1, 2, and 3 pumps and valves in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda. The licensee is committed to inservice testing in accordance with the 1983 Edition of the Code and Summer 1983 Addenda.
  - a. Section XI, Subsection IWP-3100 requires that each measured test quantity be compared with the reference value of the same quantity. Any deviations determined shall be compared with the limits given in Table IWP-3100-2 and the specified corrective action taken.

Contrary to the above, Unit 1 and Unit 2 turbine driven auxiliary feedwater pump inservice testing procedures FNP-1-STP-22.16 and FNP-2-STP-22.16 did not contain acceptance criteria for a comparison of pump differential pressure for the Table IWP-3100-2 High Values in the Alert Range and Required Action Range. As such, differential pressure comparisons were not made with the limits given in Table IWP-3100-2.

b. Section XI, Subsection IWV-3522 requires valves whose function is to prevent reversed flow to be tested in a manner that proves that the disk travels to the seat promptly on cessation or reversal of flow.

Contrary to the above, the Unit 1 and Unit 2 turbine driven auxiliary feedwater pump steam supply stop check valves HV3235A and HV3235B were not reverse flow tested in a manner that proves the disk travels to the seat on cessation or reversal of flow.

c. Section XI, Subsection IWV-3522 requires valves whose function is to prevent reversed flow to be tested in a manner that proves that the disk travels to the seat promptly on cessation or reversal of flow.

Contrary to the above, the Unit 1 and Unit 2 motor driven auxiliary feedwater pump discharge check valves VOO2A and VOO2B were not reverse flow tested in a manner that proves the disk travels to the seat on cessation or reversal of flow.

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d. Section XI, Subsection IWV-3412 requires values to be exercised to the position required to fulfill their function. Subsection IWV-3413(b) requires the stroke time of all power operated values to be measured.

Contrary to the above, Unit 1 and Unit 2 turbine driven auxiliary feedwater pump steam supply stop check valves HV3235A and HV3235B are power operated valves which perform a function in the closed direction, and stroke times were not measured.

This is a Severity Level IV violation (Supplement I).

B. Technical Specification 4.4.5.1 requires each Power Operated Relief Valve (PORV) to be demonstrated operable at least once per 18 months by performance of a channel calibration and operating the valve through one cycle of full travel. A channel calibration requires that the entire channel be calibrated, and shall include a channel functional test, in which an injection of a simulated signal into the sensor verifies operability, including alarm and/or trip functions.

Contrary to the above, PORV channel calibration did not fully test the automatic function contacts and associated wiring and circuitry. As such, the entire PORV channel was not calibrated.

This is a Severity Level IV violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, Alabama Power Company is hereby required to submit a written statement or explanation in response to Violation A only to the Nuclear Regulatory Commission, ATTN: Dociment Control Desk, Washington, DC 20555, with a copy to the Regional Administrator, Region II, and a copy to the NRC Resident Inspector, Farley, within 30 days of the date of the letter transmitting this Notice. This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) admission or denial of the violation, (2) the reason for the violation if admitted, (3) the corrective steps which have been taken and the results achieved, (4) the corrective steps which will be taken to avoid further violations, and (5) the date when full compliance will be achieved. Where good cause is shown, consideration will be given to extending the response time. If an adequate reply is not received within the time specified in this Notice, an order may be issued to show cause why the license should not be modified, suspended, or revoked or why such other action as may be proper should not be taken.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by Albert F. Gibson Albert F. Gibson, Director Division of Reactor Safety

Dated at Atlanta, Georgia this<sup>22nd</sup> day of November 1989