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November 30, 1989

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

Subject: Zion Nuclear Power Station, Units 1 and 2
License Nos. DPR-39 and DPR-48
Responses to Generic Letter 89-21
NRC Docket Nos. 50-295 and 50-304

Reference: October 19, 1989 Letter from J.G. Pantlow
to All Holders of Operating Licenses

Dear Sir:

Generic Letter 89-21 was issued as part of the NRC's continuing effort to validate staff understanding regarding the implementation of responses to Unreviewed Safety Issues (USI). One aspect of this effort is to ensure that the licensee and NRC personnel agree on the status of USI resolution implementation at each facility. This letter provides Commonwealth Edison's response, in part, to the USI's. However, some of these issues have required extensive research and the responses for those are not available at this time. Specifically, the responses to USI's A-1, -3, -12 and -43 are still under review and will be provided later. In a telephone conversation on November 29, 1989 between P. Shemanski (NRR) and G. Trzyna (CECo), it was agreed that the submittal of the additional information for these open USI's could be deferred until December 13, 1989.

The Attachment to this letter provides the status of the items in a table that completes Enclosure 1 of the Generic letter.

Please direct any further questions that you may have regarding this issue to this office.

Very truly yours,

S. C. Hernandez
for G.E. Trzyna
Nuclear Licensing Administrator

cc: Senior Resident Inspector - Zion
Chandu Patel - NRR
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ZION GENERATING STATION

ATTACHMENT

UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	ALL		Response will be provided by 12/13/89.
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	C	Internal CECO review performed in April, 1984 verified that acceptable leak detection equipment was installed & no response was necessary.
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR		Response will be provided by 12/13/89.
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	NA	--
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	NA	--
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	NA	--

- * C - COMPLETE
 NC - NO CHANGES NECESSARY
 NA - NOT APPLICABLE
 I - INCOMPLETE
 E - EVALUATING ACTIONS REQUIRED

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USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	NA	---
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	NA	--
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	I	September 8, 1989 letter from CECo to NRR transmitted implementation dates: Spring 1990 - Unit 2 Fall, 1990 - Unit 1
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	NA	
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	All	E	- December 20, 1985 letter from CECo to NRC advised that 50 ft-lb date will be 1994. This date could change as new specimens are withdrawn.
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWP		- November 13, 1989 letter from CECo to NRR transmitted recent Unit 2 specimen data.
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/77 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18 (No requirements)	All	I	Response will be provided by 12/13/89. Station response was transmitted in IPE response to GL88-20 on October 27, 1989.
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	C	November 21, 1984 letter from NRC to CECo transmitted SER First Regional inspection of program in January 1985.

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<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOP Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	C	Tech Specs were implemented on April 28, 1980. They were recently revised by Amendment No. 110/99.
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLS After 01/79.	NA	--
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C	March 22, 1984 letter from NRC to CECO transmitting SER. Phase I provided sufficient protection from dropped loads.
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SRP 6.2.1.1.C	BWR	NA	--
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CP-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	NA	Addressed in accordance with response to USI-A-46.
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	NA	--

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USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.8? (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	All		Response will be provided by 12/13/89.
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	All	E	Review of proposed modifica- tion is scheduled to be completed by June, 1990.
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	All	E	Letter submitted to NRC on October 27, 1989 in response to IPEGL88-20.
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	All	E	Generic Safety Evaluation was issued on July 29, 1988. Several plant walkdowns are pending through 1992.
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG- 1218 GL 89-19	All	E	Response due to NRC by March, 1990.
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	All, except PWRs with large dry containments	NA	-
A-49	Pressurized Thermal Shock	PGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	E	Several letters of corre- spondence have been exchanged between CECO belongs to a B&W - Owner's Group which has been established to address the issue scheduled to update NRR in February, 1990.