

U-601563
L30-89(11-27)-LP
8G.120

ILLINOIS POWER COMPANY



CLINTON POWER STATION, P.O. BOX 678, CLINTON, ILLINOIS 61717

November 27, 1989

Docket No. 50-461

Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Clinton Power Station
Response to Request for Information Concerning
Status of Implementation of Unresolved Safety
Issue (USI) Requirements (Generic Letter 89-21)

Dear Sir:

Please find enclosed the results of the Illinois Power (IP) review and reporting of the status of implementation of Unresolved Safety Issues for which a final technical resolution has been achieved.

Sincerely yours,

A handwritten signature in cursive script, appearing to read 'D L Holtzscher'.

D. L. Holtzscher
Acting Manager -
Licensing and Safety

SFB/krm

Enclosure

cc: Regional Administrator, Region III, USNRC
NRC Clinton Licensing Project Manager
NRC Resident Office
Illinois Department of Nuclear Safety

8912040062 891127
PDR ADOCK 05000461
P PDC

A012
1/1

UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*/REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	All	NC/SER Appendix C Section C.5 evaluated as no additional action required.
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	NA
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	NA
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	NA
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	B&W-PWR	NA
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	NA

- * C - Complete
- NC - No Changes Necessary
- NA - Not Applicable
- I - Incomplete
- E - Evaluating Actions Required

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*/REMARKS</u>
A-7/D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Supp. 1 GL 79-57	Mark I-BWR	NA
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Supp. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	NA
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10CFR50.62	All	C (10/85)/ SSER *6, Section 15.2.1
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	C (11/86)/USAR Section 5.3.3.1.4.5. NRC Letter March 1, 1988, Docket #50-461. ISI Manual App. 4. transmitted by IPC** letter U-600753 dated 11/7/86.
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10CFR50.60/82-26	All	C (7/82)/SSER 1, Sections 5.3.1, 2 and 3
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWR	NA
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/72 NUREG-1174, NUREG- 1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/	All	NC/GL 89-18 did not require actions of IP.

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*/REMARKS</u>
		CR-4470, GL 89-18 (No requirements)		
A-24/ MPA B-60	Qualification of Class IE Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10CFR50.49 GL 82-09, GL 84-24, GL 85-15	All	C (9/86)/SSER 5, Sections 3.11.3.2, 3.11.5. SSER 6, Sections 3.11, 7.7.3. SSER 7, 3.10.1.
A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOR Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	I/due 4/90 IPC letter U-601317 dated 12/6/88 responding to GL 88-11 identifies need for Tech. Spec. change.
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLS After 01/79	C (2/82)/SER Section 5.4.2, (Table C-1). A-31 resolved by NRC in 5/78 by issue of SRP Section 5.4.7. SER was issued in 1982; implemented prior to licensing.
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	I/(procedure changes due 9/90). SSER 5, Section 9.1.5. Additional procedural controls to be implemented prior to next refueling.

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*/REMARKS</u>
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-073,0783,0802 NUREG-0661 SRP 6.2.1.1.C	BWR	C (1/86)/ SER, Section 6.2.1.3, SSER 5, Section 6.2.1
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	NC/SSER 7, Section 3.10.1
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	I/Tech. Spec. change required (due 2/90). SSER 1, Section 5.2.3.
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, RG 1.82 (Rev. 0), SRP 6.2.2 GL 85-22 (No requirements)	All	NC/USI A-43 and GL 85-22 did not contain requirements for IPC.
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10CFR50.63	All	I/due 5/90, IPC letter U-601427 dated 4/16/89 provided the results of the IPC review on SBO. All actions scheduled to be completed by this date are complete. Additional procedure changes will be made by 5/90.

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*/REMARKS</u>
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements) GL 88-20	All	I/due 9/92. IPC letter U-601549, dated 10/27/89 (A-45 now included in Individual Plant Examination Program).
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	All	NC/SSER 7, Section 3.10.1
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG-1218 GL 89-19	All	E/(due 3/2/90)
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10CFR50.44 SECY 89-122	All, except PWRs with large dry containments	I/due 12/90 for Emergency Operating Procedures (EOP) completion. All equipment is installed. IP will submit documentation demonstrating compliance within 6 mos. of issuance of the General Hydrogen Control Program SER. IPC is developing Hydrogen Control EOP. Hydrogen mitigation procedures are in place.

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*/REMARKS</u>
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10CFR50.61/GL 88-11	PWR	NA

* SER/SSER - Safety Evaluation Report for Clinton Power Station - NUREG-0853

** IPC - Illinois Power Company