



Portland General Electric Company

David W. Cockfield Vice President, Nuclear

November 27, 1989

Trojan Nuclear Plant
Docket 50-344
License NPF-1

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington DC 20555

Dear Sir:

Generic Letter 89-21, Status of
Implementation of Unresolved Safety Issue (USI) Requirements

As requested by Generic Letter 89-21 dated October 19, 1989, attached please find the status of implementation of USIs applicable to the Trojan Nuclear Plant for which a final technical resolution has been achieved.

Sincerely,

Attachment

c: Mr. John B. Martin
Regional Administrator, Region V
U.S. Nuclear Regulatory Commission

Mr. David Stewart-Smith
State of Oregon
Department of Energy

Mr. R. C. Barr
NRC Resident Inspector
Trojan Nuclear Plant

Subscribed and sworn to before me this 27th day of November, 1989.

Notary Public of Oregon

My Commission Expires:

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PDR ADOCK 05000344
PDC

July 5, 1992

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TABLE 1 UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL
TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA Number</u>	<u>Title</u>	<u>Reference Document/ Generic Letter</u>	<u>Applicability</u>	<u>Status/Date (C)(NC)(NA)(I)(E)</u>	<u>Remarks</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, NUREG-0737, Item I.A.2.3 SRP Revisions	All	C, 12/23/80	1. PGE letter dated 12/23/80. 2. INPO accreditation of training.
A-2/ D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 NUREG-0371 GDC-4/84-04	PWR	C	NRC Safety Evaluation on Elimination of Pos- tulated Primary Loop Pipe Ruptures as a Design Basis, dated 08/05/88.
A-3	Westinghouse Steam Generator Tube Integrity	SECY 86-97 NUREG-0844/85-02 (No Requirements)	W-PWR	NC	
A-4	CE Steam Generator Tube Integrity	NUREG-0844 (No Requirements)	CE-PWR	N/A	
A-5	B&W Steam Generator Tube Integrity	NUREG-0844 (No Requirements)	B&W-PWR	N/A	

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NA - Not Applicable
I - Incomplete
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A-6	Mark I Short-Term Program	NUREG-0408	Mark I-BWR	N/A	
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661, Suppl. 1/ 79-57	Mark I-BWR	N/A	
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC-16	Mark II-BWR	N/A	
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	I, 07/90	ATWS Mitigating System actuation circuitry to be installed during 1990 refueling outage.
A-10/ B-25	BWR Feedwater Nozzle Cracking	NUREG-0371 NUREG-0619/Letter from D.G. Eisenhut dated 11/13/80 81-11	BWR	N/A	
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/82-26	All	C, 6/85	WCAP-10861, Analysis of Capsule X from PGE Trojan Reactor Vessel Radiation Surveillance Program.

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A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 NUREG-0371 NUREG-0606 SRP Revision 5.3.4	PWR	N/A	
A-17	Systems Interaction	NUREG-1174, NUREG-1229, NUREG/CR-3922 NUREG/CR-4261 NUREG/CR-4470 Generic Letter 89-18 (No Requirements)	All	E, 12/31/89	
A-24/ B-60	Qualification of Class IE Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 82-09, 84-24, 85-15	All	C	1. NRC Safety Evalu- ation dated 12/04/84. 2. PGE letter dated 01/28/85.
A-26/ B-04	Reactor Vessel Pressure Transient Protection	NUREG-0224 NUREG-0371	PWR	C, 12/06/82	NRC Safety Evaluation dated 12/06/82 (Amend- ment 78 to NPF-1).
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0371, NUREG-0606 Reg. Guide 1.113 Reg. Guide 1.139 SRP 5.4.7	All OLS After 01/79	N/A	

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 Page 4 of 5

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A-36/ C-10/ C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5/81-07, 83-42, Letter from D.G. Eisenhut dated 12/22/80	All	C, 06/24/84	PGE letter dated 06/24/84
A-39	Determination of SRV Pool Dynamic Loads and and Pressure Transients	NUREG-0802 NUREG-0661 SRP 6.2.1.1.c	BWR	N/A	
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054 NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161 NUREG-1233, NUREG-4776 NUREG/CR-3805, NUREG/ CR-5349, NUREG/CR-3509	All	E, 09/92	Only item 4 "design of flexible vertical tanks" is applicable to Trojan. The evalu- ation will be in- cluded in the evalu- ation for USI No. A-46.
A-42/ B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1/ 81-03	BWR	N/A	
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869 NUREG-0897 Reg. Guide 1.82 SRP Revision/85-22	PWR	NC	

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A-44	Station Blackout	Reg. Guide 1.155 NUREG-1032, NUREG-1109 10 CFR 50.63	All	C, 12/91	Implement procedure changes.
A-45	Shutdown Decay Heat Removal Requirements	NUREG-1289, NUREG-1292 NUREG/CR-5230 SECY 88-260 (No Requirements)	All	E, 09/92	
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/87-02, 87-03	All	E, 09/92	
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG-1218 Generic Letter 89-19	All	E, 03/16/90	
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns	10 CFR 50.44	All, except PWRs with large dry containments	N/A	
A-49	Pressurized Thermal Shock	NUREG-0649 Reg. Guides 1.154 and 1.99 10 CFR 50.61/88-11	PWR	E, 02/28/90	

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