

OFFICIAL TRANSCRIPT OF PROCEEDINGS

Agency: Nuclear Regulatory Commission

Title: Public Workshop on Technical and
Policy Considerations for Nuclear
Power Plant License Renewal

Docket No. SUMMARY OF CONCURRENT SESSIONS

LOCATION: Reston, Virginia

DATE: Tuesday, November 14, 1989 **PAGES:** 1 - 53

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UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

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PUBLIC WORKSHOP
ON
TECHNICAL AND POLICY CONSIDERATIONS
FOR
NUCLEAR POWER PLANT LICENSE RENEWAL

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Summary of Concurrent Sessions

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Sheraton Resort Hotel
Conference Rooms A, B and C
11810 Sunrise Valley Drive
Reston, Virginia

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Tuesday, November 14, 1989

1:18 o'clock p.m.

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1 SESSION LEADER:

2 Themis P. Speis, Office of Research, Deputy Director
3 for Generic Issues

4 PARTICIPANTS:

5 Robert Bosnak, NRC
6 Frank Gillespie, NRC
7 Don Cleary, NRC
8 Larry Shao, NRC
9 Milt Vagins, NRC
10 Ashok Thadani, NRC
11 James Richardson, NRC
12 John Haseltine, Yankee Atomic Electric Co.
13 Terry Pickens, Northern States Power
14 Bill Rasin, NUMARC

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P R O C E E D I N G S

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MR. SPEIS: I think we are ready to start.

Ladies and gentlemen, my name is Themis Speis. I am from the Office of Research, Nuclear Regulatory Commission. Of course, this is the last session of the workshop, and it's called Summary of Concurrent Session. The workshops we had yesterday and today, we will summarize this afternoon.

What we would like to do is call upon the chairmen of the concurrent sessions to provide a summary of most of the important issues or some of the key issues or questions that were raised during these workshops, and in addition to our people who do it for the seven concurrent sessions, the industry representatives will also provide their summary of what happened the last day and a half here.

The next step on our side is to inform our Commission of what took place here the last two days or so. They have asked us specific questions that we have to provide answers to, and of course, the other thing that is more important is to take the questions and the insights and the comments that were provided by you and start finalizing the proposed rule.

As was indicated the last two days, the schedule for the proposed rule is next spring, and the final rule is 1992. Of course, you have provided very strong comments that you would like to see the rule done a year earlier, to make sure that the rule is available when the lead plant applications

1 come in, and this is something that we will have to consider
2 very carefully.

3 As you know, the key issue that has led us to choose
4 1992 is the issue of the GEIS, the Generic Environmental Impact
5 Statement, so these are things that we have to think very
6 carefully on, and we will do the best we can.

7 We don't have in mind to provide any positions today
8 based on what we heard, it's premature. We understand also
9 that in addition to what we heard, and the viewgraphs, industry
10 will provide detailed responses to the issues that have been
11 raised, as well as the answers to the specific questions that
12 we have been able to put together.

13 So we are looking forward to those answers before we
14 finalize our positions on these important issues.

15 I think, speaking for the Staff, that this workshop
16 accomplished the objectives that we have set forth. We want to
17 make sure that you people understand our approach, and
18 understand the details of implementing this approach.

19 As most of you have said, there seems to be general
20 agreement or general philosophy on the general approach, but
21 there are differences, different views on the details of
22 implementation. And, of course, our position has been a draft
23 one, and this workshop is part of the process of going forward
24 and putting the rule together.

25 So, without losing any more time, I would like to

1 call upon the chairmen now to summarize the issues. I hope
2 that they can keep their presentations to no more than 10
3 minutes, so we can have some time for the three industry
4 representatives, as well as maybe open the floor later on for
5 some additional questions or comments. And I am sure that all
6 of you are looking forward to rushing to the airports or
7 wherever to go back to where you came from.

8 So, with that, again I thought that this was a very
9 constructive workshop, because I had the honor or dishonor of
10 chairing the IPE seminar back in Texas the earlier part of the
11 year, and it was full of fire, and I thought that Zeus was
12 coming down from the mountain.

13 [Laughter.]

14 MR. SPEIS: So, with that, I will call upon Bob
15 Bosnak to provide his summary of Sessions 1 and 5.

16 MR. BOSNAK: Yes, thank you, Themis.

17 What I tried to do here first, and we may have left
18 some important ones out, but we tried our best to include all
19 the issues, and the first slide indicates how we grouped these
20 things.

21 A are issues on which there was general agreement.
22 There were five of those.

23 B are the ones that are more difficult to fix.

24 C are the ones that perhaps are easier to fix, and
25 things that were put into categories B or C -- there was not

1 general agreement. I didn't have a chance to talk with
2 industry or the staff, so there may be disagreements on what's
3 easy and what's difficult. There's a total of 19, as you see
4 here.

5 Let's just go quickly through some of them to give
6 you an idea of where there is general agreement.

7 The current licensing basis provides adequate safety
8 level and, of course, the license renewal should focus on
9 aging.

10 Next, it is important, we think, because we want to
11 know what is necessary in the area of regulatory guidance, and
12 we did hear some comments that format and content and
13 screening, there was some guidance needed.

14 Now as far as the lead plants and their influence on
15 this whole process, we would wait for the reviews of the lead
16 plants to see if there is anything else that is needed.

17 The last bullet that you see, specifics and details
18 on Reg Guide versus the detailed rule, I thought I heard that
19 people preferred to have those in the Reg Guide. If there's
20 disagreement on those, perhaps in the written statements, you
21 can clarify that.

22 Next.

23 Now we get into the ones that are a little more
24 difficult. The current licensing basis should not be
25 documented, and you only want aging-related portions and time-

1 dependent exemptions, and we will get into those a little
2 later, because there may be again disagreements as to what the
3 staff feels are important time-dependent areas and aging-
4 related areas, and what the industry as a whole thinks of
5 those.

6 Certification of compliance is unnecessary, and
7 there's no need to define the current licensing basis in the
8 rule. Obviously the utilities felt they have a complete
9 knowledge of current licensing basis and they should make this
10 available.

11 Next.

12 Another one of the more difficult ones is with
13 respect to the conceptual approach is inconsistent with the
14 basic philosophy. The requirements with respect to components,
15 systems and structures does not give enough credit to existing
16 programs for managing aging. These were the points that were
17 made. And it is not necessary to reanalyze design of aging as
18 adequately addressed initially. It may be difficult to prove,
19 but again that was a point that was made.

20 You wanted the options, you did not want to have it
21 included in the rule to specifically identify evaluated trend
22 degradation mechanisms. You felt that this was excessive,
23 particularly if you looked at all the numbers of components.
24 There are many thousands in a given plant, and you felt that
25 this was too prescriptive for the license renewal rule.

1 Tech specs are not a suitable instrument for
2 controlling aging management was a point that was made.

3 Backfit. This is a difficult one. The rationale for
4 the backfit rule, I think the point was made that it's valid in
5 relationship to license renewal issues, including aging
6 concerns; that if an aging concern is valid, you should go
7 through the benefit and cost relationships to show that that is
8 a valid point, and the backfit rule again has the built-in
9 procedures that if you were talking about a minimum level,
10 adequate protection, it's already included.

11 There was an additional point about ambiguity in the
12 backfit rule and the conceptual approach, and that should be
13 corrected.

14 Maintenance requirements, you felt that they should
15 not serve to replace a maintenance rule, and any requirements
16 put in the license renewal area should relegate itself to aging
17 degradation. We should accept current practices, supplemented
18 as needed for aging management, to support the license renewal,
19 and aging management methods should not be prescribed in
20 detail. Those were the points, again, and they are important
21 because maintenance is a vital thing with respect to managing
22 aging.

23 Now we are getting into the last category, and some
24 people may disagree with respect to easy to solve or hard to
25 solve, but one of the points that you made on severe accident

1 closure was that there is an existing generic letter out, and
2 it should take care of the severe accident area, and since
3 severe accidents are not a consequence of aging -- although
4 some people might want to debate that -- the current licensing
5 basis should be defined as including treatment of severe
6 accidents. That was a point that I think was made by Joe
7 Gallo. And the accident management programs are being
8 addressed. The NUMARC working group has guidelines for
9 evaluating accident management capabilities.

10 The rulemaking schedule, I think, was one of the last
11 things that I wanted to mention, and you have heard about that
12 again with respect to mid-'91 for the lead plants, but if we
13 are going to couple environmental and technical issues, it may
14 go as far as April 1992.

15 Now into an area just very quickly of definitions,
16 and there is a group from NRC and EPRI that are working on the
17 time-dependent processes. The issues that are important to
18 safety, I think the point was made that there is an existing
19 definition of important to safety. Even though the one that is
20 in the current rule is clearly stated, there could be some
21 problems if people don't understand what important to safety
22 really means.

23 And finally, the last thing, the current licensing
24 basis, what we are trying to say here, that there are important
25 things that go into age-related degradation mechanisms. If we

1 don't agree on what they are, and that's the last bullet on the
2 slide, we have indicated nine identified mechanisms -- if we
3 don't agree on what they are, then we're going to have problems
4 in trying to decide what to do with respect to the current
5 licensing basis.

6 And the properties and parameters that change with
7 time, we ought to agree on what vital properties and parameters
8 change with time. So there is work going on with that. It was
9 an area that was perhaps not discussed in the general sessions
10 as much as in the specific technical sessions.

11 Thank you.

12 MR. GILLESPIE: I want to add two things to what Bob
13 said, because I think in two different sessions, we went round
14 and round and round the horn on this. I try to say this
15 without getting into too much trouble. I've been advised by
16 legal counsel not to do it but I'm going to give it a try
17 anyway.

18 One of the links -- in the wording of the rule, we
19 may have to go back and we'll take a look at it -- but one of
20 the links between the existing license and this thing, this
21 renewed license, I won't call it anything else than renewed
22 license, it's that ill-defined, is a listing of what the
23 current obligations are. As those obligations get carried
24 forward, so do the original findings and it's an important link
25 to make.

1 If you look in the standards, in the conceptual rule
2 right now, you'll find in the standards that we're not remaking
3 the original findings. So one reason for having and I'll call
4 it a list, a list of what it is as a minimum, is to make that
5 vital link to also carry forward we hope, the findings.

6 Now, that's what we're trying to do. That's our
7 intent. I don't know if we'll be fully successful or not but
8 that's the intent of making the link. So, I was asked to at
9 least clarify that and that might be helpful.

10 The other thing was in the screening -- in the rule
11 -- our rule has been described as not like a funnel like people
12 would like to see it but more like a trough or a toilet --

13 [Laughter.]

14 MR. GILLESPIE: -- where everything gets flushed down
15 or nothing gets flushed down. One of the things that I think
16 we would at least be receptive to some comments on is how to
17 integrate some screening classes, I'll call them, the classes
18 which are very, very well defined into the rule itself,
19 realizing that it never can be all inclusive but it's a middle
20 ground between a prescriptive and a broad rule we currently
21 have now which causes certain analyses and information to be
22 collected and decisions made on every component.

23 The example which came out in one session which is
24 one of the easier cases is, if a component has a fixed life and
25 we basically have fixed that life because we know everything or

1 think we know everything that's going to happen to it over that
2 time, then why should we have to do an analysis of that piece?
3 Well, that's one class which we'd have to think about it more
4 but it's a good comment. It's not currently in the rule and if
5 there's other classes of things like that which are very
6 definitive, very deterministic in nature, I think we'd be happy
7 to entertain those kind of comments as far as coming to a
8 middle ground in the screening process in the rule.

9 Those were and/or slots. I want to get Don off the
10 hook because Level III PRA came up in several sessions and I
11 guess it came up most in the environmental sessions. I'm going
12 to let Don address it there. The general concepts of, should
13 the rule mandate a PRA, as Bob said his slide came across as
14 no, it shouldn't mandate it.

15 For other reasons not on the technical portion of the
16 rule but on the environmental portion, Don has to deal with the
17 question of off-site doses and some people relate a Level III
18 PRA one for one with off-site doses and I think where Don's
19 going to be coming from is maybe that isn't a one for one
20 relationship. You maybe can address off-site doses without
21 necessarily going through all the steps of a Level III PRA.

22 For the technical portion, we did not anticipate at
23 this time in the conceptual part, of requiring a PRA.

24 MR. SPEIS: The next speaker is Don Clearly who will
25 discuss the environmental issue. That is the issue that

1 received considerable discussion, dialogue, views, and based on
2 what we heard, at least wishing that you do understand our
3 position because as I said earlier, one of the objectives of
4 the workshop is to make sure you understand where we're coming
5 from.

6 So, Don, we thought, let's go into that issue and
7 then we'll proceed to the technical issues after Don is through
8 with his presentation.

9 MR. CLEARY: We heard a number of points being made
10 addressing the questions that had been put out in the
11 environmental area. Our impression was that there seemed to be
12 pretty much of a consensus. We didn't hear differing opinions.
13 We heard that a new age had begun for the license renewal rule,
14 that the schedule for that rule should be accelerated so that
15 the final rule is published in May of 1991 and this is tied to
16 the lead plant applications.

17 We heard that a generic NEPA study is either okay or
18 there are some merits to it. We didn't hear any negative
19 comments on it. That generic study however should be decoupled
20 from the Part 50 rulemaking. We heard the encouragement that
21 in developing a generic NEPA study that the staff should be
22 very careful to build on existing policies and documents that
23 have been generated over a number of years within NRC, for
24 example, in the waste confidence area.

25 We heard relative to the question of Level III PPA,

1 an acknowledgement that off-site consequences of severe
2 accidents have a legitimate place in the NEPA analysis and that
3 these off-site consequence analyses are tied or can be tied to
4 Level III methodologically in that they address the same thing
5 and that many of the tools are the same, or several of the
6 major tools are the same.

7 However, we did not see support for the feedback from
8 these off-site consequence analyses to the decision process on
9 the safety side. We also heard an offer from NUMARC to work
10 with the staff in developing information that would go into a
11 generic study and we look forward to hearing more hopefully in
12 the responses in the next two weeks as to areas where NUMARC
13 and its members might be prepared to provide assistance.

14 This is as we develop our work plan for the GEIS
15 assuming that we get the go ahead from the Commission. We
16 certainly will open this prospect up for further discussion
17 with NUMARC and with any other source of information.

18 Relative to the question on sources, both
19 environmental effects, this wasn't addressed in any detail but
20 we did hear several individuals make the statement that there
21 shouldn't be any significant effects, that whatever effects
22 there might be have been pretty much bounded by experience
23 already.

24 Relative to alternatives, we did hear the advice that
25 alternatives be carefully defined in terms of a sharp

1 definition of exactly what the Federal action is that the NEPA
2 analysis is supporting. I think those are the important points
3 that we got out of the discussion in Section 8.

4 MR. SPEIS: Thank you, Don.

5 Now we're going to get into the detailed technical
6 issues and the next speaker will be Larry Shao who will discuss
7 the primary boundary.

8 MR. SHAO: We had very discussion yesterday in
9 Session 2. Reactor pressure boundary. It was agreed that the
10 reactor pressure boundary must have very high reliability for
11 the operating life of the nuclear power plants. The key
12 components in the reactor pressure boundaries are reactor
13 vessel, steam generators, pipings, pumps, and valves.

14 The key aging mechanisms for these components are
15 radiation, fatigue, erosion, corrosion, wear, and
16 embrittlement. The seven prepared questions we discussed in
17 great details. It was concluded that seven code groups such as
18 ASME, ASTM, are looking at issues of reactor vessel safety
19 programs and currently ISI and IC programs for license renewal.

20 For aging of cast stainless steel, it was felt that
21 there were no sufficient data and more work needed to be done
22 in this area. However, the experience of cast stainless steel
23 can be evaluated from our service components. Mechanical
24 analysis can be used to resolve this issue.

25 In the ASME code -- need to be considered. These

1 curves are being developed by ASME and PVRC. The use of
2 exotransients in lieu of these X-rays is likely to reduce the
3 fatigue usage factor. -- in the area of cancellations will
4 also reduce fatigue usage factors.

5 For wear overlay repair, it was recommended that
6 improved techniques should be developed and the repair pipings
7 should be properly analyzed for the effects of wear overlay.
8 It is agreed that a new criteria could be developed. It is
9 very important that the full experience should be utilized as
10 much as possible.

11 There was also interesting discussion as to how to
12 tackle the issue related to areas that have no detailed fatigue
13 analysis. There were three options. The first option is to
14 perform reanalysis. The second is use analysis and inspection
15 results for similar plants and the last one is use fatigue
16 monitoring.

17 In addition to NRC staff, several presentations were
18 made by the industry groups. The industry groups have prepared
19 generic technical reports for PWR and BWR reactor vessels and
20 primary pressure boundaries. These reports were addressed all
21 age-related degradations. Each licensee has to show how these
22 generic reports are applicable to his plant. For areas of the
23 plant that are not properly covered by these reports, plant-
24 specific analyses had to be performed for these areas.

25 MR. SPEIS: Thank you, Larry.

1 The next speaker is Milt Vagins who will address the
2 section on fluid and mechanical systems.

3 Milt.

4 MR. VAGINS: Of course, part of our session in
5 reality was covered by boundary components. You can't have
6 fluid mechanical systems that don't include boundary components
7 but we stuck more or less to things like pumps and valves and
8 things not generally thought of as the primary boundary.

9 We start off by -- we had seven questions but the way
10 the answers came in, the way the comments came in, didn't
11 generally follow those questions. However, one thing that
12 became very clear is that clarification, at least in our area
13 was necessary to assure that the philosophy of the rule was
14 kept into or was involved in the details. I think you've heard
15 this before and you'll probably hear it again. The purpose of
16 the proposed rule is to maintain safety of current licensing
17 basis, in other words, assured continued safety -- not to
18 enhance safety.

19 This is what we've been saying all day long. One of
20 the questions raised was should we require hot functional
21 testing, benchmarking, et cetera, for license renewal and the
22 comment again was that most components are tested, inspected,
23 repaired, refurbished, or replaced under NRC programs that are
24 effective now and will continue to be effective during extended
25 operating term. Additional requirements are not needed for

1 license renewal. Understood, of course, that involved here
2 were the words, that items not affected or not showing aging
3 degradation. In other words, this is true, but there are areas
4 where aging degradation which is not or may not be a principal
5 concern now could very well be a principal concern in the
6 license renewal period.

7 Of course, pressure boundary degradation needs to be
8 assessed and that was covered by Mr. Shao, so we'll continue on
9 the next one.

10 What equipment should be included for license renewal
11 review? Here we have some very succinct statements. The only
12 components important to safety with unresolved aging
13 degradation issues as stated in philosophy. The kicker there
14 was definitions of importance to safety and it's obvious we're
15 going to have to do some more work to be sure that we have the
16 definition well pinned down and well-defined and consistent
17 throughout the rule.

18 Section XX-9 is inconsistent with philosophy because
19 it requires detailed information for all equipment important to
20 safety. The philosophy again states that we are really only
21 concerned with those equipment which show aging degradation
22 which are not handled by present licensing basis. The third
23 item was that screening is essential to allow focusing
24 resources on important issues.

25 I think of course, there's general concurrence on

1 that. That's exactly what we want screening for. We don't
2 want you to waste time on issues that are not important. There
3 becomes a basis -- a point -- to agree or disagree as the case
4 may be on what are important issues.

5 The inclusion of balance playing components should
6 follow same criteria as screening rule. The issue was brought
7 up about initiators and it was brought up in several of my
8 sessions and I'm sure in other sessions, too. Again, it's
9 something that's going to have to be examined very clearly. We
10 certainly don't want to be looking at all initiators of balance
11 of plant but we certainly do want to be looking at balance of
12 plant components that may prevent or mitigate safety systems
13 from operating if they fail.

14 We certainly don't want the old assistance to be
15 challenged more than they are now. So again, some further
16 points of analysis is required. There was a definite
17 disagreement on inclusion of components that increased
18 frequency of accident initiators. This is the point I was talk
19 I was talking about. The initiators -- some people felt that
20 initiators -- looking at these subclasses of initiators were
21 extremely important and the general feeling of the audience was
22 that initiators fall outside the bounds of present day
23 licensing basis and really shouldn't be involved because that's
24 what the safety systems are for, the backup systems, et cetera,
25 to take care of the initiators.

1 Then we moved on to the next one. Of course, the
2 question arises, do we need augmented inspection and/or
3 analyses for aging pumps and valves and this really became
4 involved with today-tomorrow problems. We all know that there
5 are weaknesses in the ISC program but we're working on it. So
6 comments said that augmented needs will be developed as part of
7 an evaluation procedures for selected components. The lead
8 plants are working specifically on that. They did feel that a
9 good part of the inspections will be handled by the IFC's and
10 that the O&M committees on ASME, the NRC, various groups, are
11 working on improving IFC's and those are today's problems.

12 The issue of trending came up in the same area and
13 most comments were, use trending only where absolutely
14 necessary and where they're effective. The issue was brought
15 up that if you have something that is due to replacement or
16 refurbishment at a periodic period and that periodic period was
17 shown to be effective, in other words, no increased failure
18 rate, then you don't need trending. It's hard to take issue
19 with that.

20 The next question was should functional or proof
21 tests be required as a prerequisite. This is a point I raised
22 before and it goes back to the general clarification and again
23 it says, current in-service testing is sufficient. If we have
24 problems with in-service assessing, let's settle them today --
25 let's handle them now.

1 The question of what requirements be for including
2 operating history, water, environment, and higher temperatures
3 of fatigue evaluation, you've heard that addressed in the
4 boundary components sections and the same goes here. Very
5 briefly, ASME is working on it. NRC is working on it. Other
6 people are working on it and again, I don't see that as
7 tomorrow's problem.

8 The next question is tomorrow's problem and that is,
9 how should the life of ASME Class II and III components be
10 determined? The general comments there were that the two and
11 three components do not normally experience severe thermal
12 cycles and the load cycles are not severe and therefore, the
13 original acceptability criteria using ASME directions were
14 sufficient. However, we brought up that in cases where a water
15 hammer has been identified and water hammer has been an issue,
16 then special analysis has to be attributed to that.

17 I'm not quite sure what that analysis will be but
18 certainly something will be necessary in that area.

19 Final question, I believe, right, is additional
20 guidance needed? Do the utilities have all the guidance they
21 need to do a good job in license renewal right now? The
22 general comment was no, no new regulatory guides are needed but
23 after issued, they should take advantage of the NUMARC NUPLEX
24 the industry technical reports and absolutely should not delay
25 -- there should not be a reason for delaying evaluation and

1 license renewal for the lead plants.

2 Again, I'm not quite sure we totally agree with the
3 fact that guidance is needed but those were the comments that
4 were made. Thank you.

5 MR. SPEIS: Thank you, Milt.

6 Ashok Thadani from the Office of Nuclear Reactor
7 Regulations will discuss the screening methodology for systems,
8 structures and components, and you might as well stay here to
9 discuss the electrical systems.

10 MR. THADANI: Actually I think Milt has covered
11 everything, but since I made these notes, I might as well go
12 through these.

13 The screening methodology session was a fairly lively
14 session, not quite close to the IPE workshop, but it was
15 somewhat lively.

16 There were three formal presentations. NUMARC and
17 Yankee Atomic made presentations on the proposed screening
18 methodology. They indicated that there is a basic
19 inconsistency between NRC staff philosophy and the explicit
20 requirements of the proposed rule and gave several examples.

21 Bill Vesley of SAI described his work on the use of
22 PRA methods to quantify the impact of aging on component
23 failure rates, the effect of multiple aging mechanisms, and the
24 effect of maintenance and surveillance programs on failure
25 rates.

1 These three formal presentations led to a fair amount
2 of discussion, and let me give you a sense of some of the
3 issues that people raised.

4 First, the industry consensus seems to be that the
5 information requirements in 9(c) are in fact excessive. Milt
6 also touched on that. They believe that the required
7 information should be related to the results of the screening
8 process, and on different aging management strategies that are
9 chosen.

10 They also indicated that the focus of the screening
11 process should be on structures, systems and components, which
12 are important to safety as defined in the licensing basis.

13 This initial scope of the systems, structures and
14 components should then be reduced, based on existing programs
15 and those not subject to significant age-related degradation.

16 Only the remaining components which have unresolved
17 age-related degradation issues should be required for further
18 evaluation.

19 There was a fair amount of discussion on the
20 inclusion of balance of power equipment that could lead to
21 transients. There seemed to be consensus that it's okay to
22 include balance of plant equipment if it's utilized to mitigate
23 transients, but not necessarily as the initiating equipment.

24 There seemed to be a general sense that the utility
25 that embarks on license renewal activity should have a pretty

1 good handle on design basis information.

2 There was concern that not enough credit, at least
3 the language in the conceptual rule doesn't give enough to the
4 established programs to mitigate aging-related degradation.

5 Bill Vesley raised some questions about whether the
6 methodology proposed by NUMARC is able to deal with multiple
7 component failures due to aging and the impact on that failure
8 rate.

9 PRAs can be used to evaluate aging effects and
10 effectiveness of maintenance programs and controlling core melt
11 frequency, and are useful in prioritization. The key word there
12 is prioritization.

13 The NUMARC methodology is based on a deterministic
14 approach with PRA used to augment and provide additional
15 insights, but the sense was that PRA should not be mandatory.

16 Those are some of the highlights of Session 4.

17 Now let me go to Session Seven, which is the
18 electrical systems.

19 Again there were three formal presentations during
20 this session. NUMARC discussed the approach to life extension
21 of electrical components, with specific emphasis on cables and
22 containment.

23 Yankee Atomic commented on the appropriateness and
24 scope of Section 9 again, and that kept coming up again and
25 again, and I think in all the sessions.

1 Bob McCoy of Yankee Atomic also addressed the
2 benefits of existing programs that apparently are not
3 recognized, and indicated that Section 9 should incorporate a
4 screening process and only look at equipment not in the
5 existing programs, and only equipment subject to significant
6 aging degradation, one morpe time.

7 Mr. J.B. Gardner, consultant, gave some comments on
8 the rule and some of the other issues he was concerned about,
9 and then let me give you again the flavor of the type of
10 comments that were made.

11 There is, one more time, inconsistency between the
12 proposed rule and the staff philosophy, and there was a
13 question in terms of what should be included under important to
14 safety. A discussion on how credit will be given for existing
15 programs; how much description of these programs should be
16 provided in submittals.

17 There was a fair amount of discussion, but not real
18 answers, I would say, to what are considered the now issues,
19 and let me clarify what that means. Some people in the
20 audience said that we know of a number of problems with
21 electrical systems, and how are these problems going to be
22 treated in the license renewal process? And questions came up,
23 are those problems that should be dealt with right now, because
24 they are problems today, and what is the significance of these
25 problems for license renewal activity? Lots of discussion; no

1 answers.

2 Some people, of course, feel that today's problems
3 should be dealt with today, and should have nothing to do with
4 the license renewal process.

5 Generally the industry feels that no additional
6 requirements should be imposed on systems that are covered by
7 the EQ rule.

8 There also seemed to be a common view that the
9 effectiveness of the EQ program for older plants may need a
10 further look, if one were to go through the license renewal
11 process for those older plants.

12 Redundancy. There was a fair amount of discussion.
13 What does one mean by important to safety? Redundancy is there
14 to provide defense-in-depth, and the general sense was that
15 even failure of a single crane, if caused by aging, should be
16 given close attention.

17 There was a fair amount of discussion, one more time,
18 on whether the equipment that could lead to transients should
19 be included in the scope or not, just as it was discussed in
20 the screening session.

21 Gardner raised some questions. He said, you know,
22 this research shows good and research shows bad, and he said if
23 we utilize research, things are actually better than we thought
24 they might be and take credit for that, then we should also
25 recognize that research is showing that there may be problems

1 where we didn't realize there were problems, and we ought to be
2 focusing attention on that. I think he was looking for some
3 balance in the way we look at the issues as we go through this
4 process.

5 There was some discussion of what is meant by
6 trending and what needs to be trended, but a comment was made
7 that was sort of interesting that -- again it was Gardner, I
8 think, who said he had looked at some data and found that the
9 root cause analysis was not very good, and he says this root
10 cause analysis is not very good, and what are you trending?
11 Good question, I think.

12 Also another comment in the area of electrical
13 systems, we focused a lot of attention on cables. The point
14 was made that we should be careful when we talk about cables,
15 we should really be talking about cable systems. This includes
16 connectors and so on, the whole thing, and not just cables.

17 Again the session was quite effective in surfacing
18 some of the major issues, I think.

19 Thank you.

20 MR. SPEIS: Richardson will now give the last
21 technical presentation from the staff side on the session on
22 containments.

23 MR. RICHARDSON: Session 6 dealt with containments.
24 We also had three formal presentations by industry.

25 The first presentation, by Bob Nichol, representing

1 NUMARC, noted that there are three industry reports coming
2 forth in the area of containments and structures -- one on PWR
3 containments, one on BWR containments, and the third on Class I
4 structures, and he also noted that these industry reports will,
5 in general, make generic conclusions, and then it will be up to
6 each of the licensees to show that those conclusions are, in
7 fact, applicable to their individuals plants, but it was Bob's
8 conclusion that containments, in general, continue to provide
9 the structural and leak integrity necessary and that there is
10 no real concern about corrosion of tendons, or reduction in
11 tendon pre-stress, that cannot be detected over the course of
12 time, and that fatigue of expansion bellows, which is of
13 concern to people, should be able to be handled by normal
14 fatigue analysis.

15 He also noted that there is a need, however, to
16 manage degradation mechanisms in the area of acid attack,
17 particularly associated with acidic soils and ground water, for
18 those structures that are below grade; also noted a concern
19 regarding floor liner, plates that are beneath the floor slab,
20 attacked by corrosion; and finally, a need to pay attention to
21 interior coatings.

22 Tim Bailey, from Northern States Power made a
23 presentation representing the MARC I containment at Monticello,
24 and again, Tim concluded that the MARC I containment is
25 expected to continue to provide structural and leak integrity.

1 Their extensive examinations, to date, have shown no
2 indications of any substantial degradation. They have
3 estimated that their dry well is good for in excess of 100
4 years; the vent line and vent header good for at least 76
5 years; and the bellows -- vent line bellows good for something
6 on the order of 95 years; suppression chamber shell something
7 in excess of 100 years.

8 Joe McCumber, from Yankee Atomic, identified what
9 they felt were to be the significant issues in the containment
10 area, and that is liner corrosion; coating degradation, again;
11 boric acid-induced degradation; and vibration-induced
12 structural damage.

13 He pointed out the importance of walk-downs and the
14 valuable information that can be obtained from a good walk-down
15 and that existing programs need only be augmented, as
16 necessary, when appropriate degradation mechanisms are
17 detected.

18 To repeat a song that has been sung in, I guess, all
19 of the sessions, Joe commented that XX9 goes beyond the basic
20 philosophy. One gets the idea that this may have been
21 orchestrated --

22 [Laughter.]

23 MR. RICHARDSON: -- and unnecessary.

24 We need to focus our resources on the real problems,
25 and the rule needs to better define the screening process. It

1 needs to be comprehensive, clear, consistent, efficient, and
2 flexible. Sounds like the Boy Scout oath.

3 The belief we heard was that, in general, the
4 degradation mechanisms associated with containment and
5 structures are pretty well understood.

6 In general, there was comments in response to some of
7 the NRC questions that the NRC itself needs to do a better job
8 and perhaps get off its collective duff and endorse IWE and
9 IWL; that they are, in fact, being modified and expanded to
10 include inspection of the base material and that, in general,
11 these consensus standards will do a pretty good job in
12 detecting aging mechanisms.

13 We heard that we ought to pay attention, take note of
14 the testing process itself. In some cases, it may be
15 challenging the containment to the point of degrading it, and
16 it's something we need to pay attention to.

17 It was also noted that there are no real NRC
18 requirements for inspecting Class I structures, and maybe that
19 needs to be remedied.

20 From time to time, people take core samples of
21 concrete. Maybe we're not taking advantage of the information
22 that those core samples may provide and that we ought to devise
23 some programs or methods for taking advantage of these concrete
24 core samples when they're extracted, and again, as was pointed
25 out, the industry reports that are forthcoming will address the

1 need for additional inspections and tests in the concrete and
2 steel structures, and the bottom line is that the feeling we
3 heard was that there is no real need for rebaselining the
4 containment.

5 MR. SPEIS: Thank you, Jim.

6 I assume, Jim, that since NRC's Joe Scinto is not
7 here, what you meant by Joe -- Joe Gallo? When you said Joe
8 said something.

9 MR. RICHARDSON: No, it wasn't Joe.

10 MR. SPEIS: Oh.

11 VOICE: Unfortunately, Scinto is here.

12 MR. SPEIS: Oh. My apologies. My apologies. Oh did
13 I put my foot into the --

14 [Laughter.]

15 MR. SPEIS: -- into my pocket.

16 Joe Haseltine, of the Yankee Atomic, will give us a
17 summary of his perspective of what happened here.

18 MR. HASELTINE: It's with some intrepidation I get up
19 here, because I may repeat something somebody might have said,
20 but I would like to summarize, from Yankee's point of view,
21 what we consider the important points made during the last day
22 and a half, starting with the current licensing basis.

23 As you have heard, we think it should be limited. At
24 the most, we think we should provide a listing of the documents
25 that are associated with the system structures and components.

1 It should be analyzed only to the extent that you need to for
2 time dependency and for aging of the SSCs.

3 In the screening area, we believe you should only
4 supply the design information that you need to do your aging
5 evaluations and to do your screening.

6 A total list of degradation mechanisms, which a
7 comprehensive list is probably not worthwhile to try and to put
8 together, because somebody's always going to dream up a new
9 one, and it should be, really, on a component-by-component
10 basis.

11 In the maintenance area, we need flexibility so that
12 when we go through the process, those components that don't
13 have any degradation or that are covered by existing programs
14 can be put aside, and those that need a detailed evaluation can
15 have a detailed evaluation. We shouldn't have to treat them
16 all the same.

17 In the trending area, we obviously don't have to
18 trend every component. Again, it should be on some type of
19 component-by-component basis.

20 Schedule -- we want to see the proposed rule issued
21 next May. We want to see the final rule issued 1 year after
22 that. In order to do that, obviously you've got to get into
23 the environmental area, and we believe that you should perform
24 an environmental assessment to support the part 50 rulemaking,
25 consistent with the issuance of the final rule in 1991.

1 We also need an administrative change to part 51.20
2 in the same timeframe.

3 We need a substantial change to part 51 before the
4 lead plant SERs are issued in 1992.

5 So, it's a parallel path, and that's how we believe
6 you can accomplish all objectives here and get to 1992 with
7 everything in place.

8 As far as severe accidents, we don't believe there is
9 a place for it in the rule. It is going to be part of our
10 current licensing basis, and if you have to put it anywhere, we
11 think it should be in the statement of considerations.

12 PRA -- we do not believe that a PRA is needed for
13 screening -- you can do it deterministically, but we do believe
14 there's a lot of merit in using it individual component
15 evaluations.

16 The Level III PRA, we don't believe is needed in the
17 aging evaluations, since we're only interested in degradation
18 mechanisms.

19 The backfit rule should be applied to the development
20 of the renewal rule and especially in the review of Yankee's
21 application.

22 [Laughter.]

23 MR. HASELTINE: Earlier, in the first session, we
24 talked about those programs which should not be subject to a
25 review, and I think it was Frank asked whether some other

1 programs should be omitted, and we have a few here.

2 We believe that the station blackout should be
3 omitted, as well ATWS, hydrogen control, decommissioning, fire
4 protection, those general design criteria which are not subject
5 to age-related degradation, and those siting criteria which are
6 not subject to age degradation.

7 In summary, we think the NRC has done a lot of work,
8 has made a lot of progress, and you should be commended. The
9 opportunities for interactions such as this workshop have been
10 very helpful and we think they should continue.

11 Thank you.

12 MR. SPEIS: Thank you, Joe.

13 Now, Mr. Pickens, from Northern States Power, will
14 provide his perspective through his summary.

15 MR. PICKENS: Thank you, Themis.

16 Actually, I'm going to play a dual role here. Based
17 on some input that we received last night -- or feedback --
18 primarily from people who were in the Structure, System, and
19 Component Evaluation Section, we just wanted to very briefly
20 run through some of the key points that we made on each one of
21 the steps and that is to go into a little bit what it is as we
22 go through the screening that we intend to provide to
23 disposition components and structures and the type of
24 information that we think is necessary in the application.

25 In the first step, Step 1-A, let me first go back.

1 It's a two-level evaluation. There's a system level evaluation
2 and once we've gone through the system-level evaluation, we
3 take the systems that remain that have not be dispositioned and
4 we put them through a component-level evaluation. I think it's
5 worth stressing that we feel we are actually doing evaluations
6 -- not just screening. We're not just taking things out and
7 putting them out there and kind of leaving them in never, never
8 land. We are going to at each step of the process provide
9 documentation as to why it was appropriate and how we drew
10 those conclusions for the NRC's review in the application.

11 Step 1-A, very quickly, just gets into taking all
12 plant systems, putting them into 1-A and really looking at
13 those that are relied upon to operate safely. We believe that
14 the list of systems that we've come up with, the way we've
15 defined the criteria, comes very close to giving us that list
16 of systems that is important to safety and we didn't have a lot
17 of conflict or discussion on that point yesterday during that
18 session.

19 In Step 1-B, we are going through and we are
20 identifying the systems and structures that significantly
21 affect the radiological health and safety risk to the public.
22 To be dispositioned from further evaluation at this step, we
23 are going to document what the system function is and the
24 conclusion that it does not affect the radiological health and
25 safety risk to the public and the basis on which we've made

1 that conclusion. Again, this will be part of the application
2 so that the NRC can review the basis of our conclusion and how
3 we're dispositioning it at this point.

4 For those systems which are dispositioned at this
5 point and throughout all the steps as we go through, that's
6 where we come from when we say we don't need to provide the
7 level of information that's called for in 9-C of the conceptual
8 outline.

9 Step 2-A, Step 2 is where we get down to the
10 component level evaluation. This is where we're looking for
11 those components in a system which do not contribute to
12 performing a safety function or could prevent or preclude the
13 system safety function from being performed. To be
14 dispositioned, the industry will document that a component is
15 not necessary for a system to perform the safety function and
16 conversely, that its failure also would not preclude the system
17 safety function from being performed.

18 At Step 2-B, we are going to be getting into
19 reviewing components that are subject to established effective
20 replacement, refurbishment, or inspection programs -- credit
21 for existing programs that we keep talking about. To be
22 dispositioned from further evaluation at this step, the
23 industry approach would document that the components' safety
24 functions, the degradation mechanisms which could preclude
25 those components from performing those safety functions and the

1 programs which ensure the function is maintained.

2 So step by step component degradation mechanisms and
3 the programs and how they effectively deal with those safety
4 functions and the degradation mechanisms.

5 At Step 2-C, we're going to identify those components
6 which are subject to significant age-related degradation, or,
7 to put it another way, we're going to look for existing,
8 established, documented sources that have precluded that
9 there's significant age-related degradation that would affect a
10 component. To be dispositioned from further evaluation at this
11 step, we would reference the documents which provide the basis
12 for the conclusion that the component is not subject to
13 significant age-related degradation or with the risk criteria
14 that are included, we would provide the risk assessment results
15 which conclude that even if this component went to failure,
16 that the risk increase would not be significant.

17 Finally, at Step 2-D, this is the components which
18 have been identified, which are subject to potentially
19 significant age-related degradation and if were allowed to
20 occur unmanaged, could affect safety. We've listed various
21 options, by no means all-inclusive in the document, as examples
22 of different ways that we can address it. Depending on the
23 option that you pick to address, the significant age-related
24 degradation which might affect that component, there's varying
25 degrees of information under 9-C which would need to be

1 provided. There may be some which you can just say, I know
2 enough to say that this replacement interval is good. You
3 provide the basis for that. Others where you may want to do a
4 complete technical evaluation to justify that the aging
5 mechanism, that you understand it, that you don't want impact
6 at all over the life. You need to put that forward. In that
7 case, you're going to need a lot more information -- design
8 basis, environmental conditions -- all that type of
9 information.

10 So it varies, but at each one of these, we would
11 submit and as part of our documentation for the lead plant,
12 either reference in our application or supply it, the
13 information that would justify it through the screening
14 process. I guess that concludes the comments that I want to
15 make for the NUMARC screening methodology portion. Now, I
16 don't need that anymore. Now, I'll go through the NSP
17 comments.

18 I guess first Northern States Power would like to
19 express its appreciation for the opportunity to participate in
20 this forum. I think that it's a very important issue to talk
21 about license renewal. It's something that Northern States
22 Power has been very interested and active in and I guess I'm
23 very happy to see the progress that we've made over the last
24 six months to a year.

25 I think NSP's commitment in this area is evidenced by

1 the participation both as a pilot study and now as a lead plant
2 to demonstrate the license renewal process and I think beyond
3 that, it's worth noting too that at Prairie Island, we have
4 already initiated activities on plant life extension and
5 license renewal. I think there was a question earlier about,
6 is there going to be some lull after the lead plants come in
7 and I think our response to that is no, we're already planning
8 on Prairie Island, that shortly after we complete the
9 Monticello license renewal process, we'll be coming in with our
10 Prairie Island application.

11 By way of summary on the positions taken by NSP over
12 the last day and a half, we believe the NRC's philosophy stated
13 in the notice of this meeting is sound and that it provides a
14 more than adequate basis to find that the already-existing
15 adequate level of safety will be maintained during the renewal
16 period. I think the important part now is to take that
17 philosophy and the conceptual outline which we've commented on
18 and now take that and to develop it into a regulation that
19 while it ensures consistency of application, as all utilities
20 go through this process, it also provides some amount of
21 flexibility, recognizing that by the process that we've
22 undergone as a way we are licensed, there are a lot of
23 differences between utilities that are out there now.

24 So we're going to need some flexibility in addressing
25 the aging phenomenon. NSP believes that the approach is

1 outlined by the industry through the NUMARC NUPLEX working
2 group satisfies both of these principles. It gives the NRC the
3 information that it needs. It's going to provide consistency
4 of application, yet it also provides a certain amount of
5 flexibility for us to meet those requirements.

6 It provides a systematic approach in assessing the
7 structures, systems, and components which ensures the ability
8 of the plant to operate safely. In many areas, we are going
9 beyond those systems which were considered as part of the
10 original licensing basis into the balance of plant systems.
11 I've heard a number of comments about balance of plant
12 initiators, which ones are we looking at, which ones aren't.

13 We have tried to be careful through the approach that
14 we've gone through on the screening, to look at and be smart
15 about which balance of plant systems need to be included and
16 indeed, in many cases, the major balance of plant systems which
17 can be initiators are also being picked up as important to
18 safety support systems which support safety-related systems.

19 For those systems which are just balance of plant
20 initiator, Northern States Power believes that it's more of an
21 economic issue for utilities. It's certainly not something
22 that we're going to ignore. It's something that we are paying
23 attention to throughout our overall plant life extension
24 program and in addition to that, there are existing regulatory
25 oversight mechanisms such as licensee event reports,

1 performance indicators -- different things which cause us to g
2 down and look at root cause and as we identify those types of
3 things, we follow through on them and if we start seeing things
4 that come down to increased failure rates from age-related
5 degradation causing an increase in BOP initiators, we're going
6 to take a look at it and make sure it's taken care of.

7 We don't think that it needs to be addressed in the
8 definition of Important to Safety BOP Initiators, that is,
9 being a reason in and of itself to include it in for a license
10 renewal review.

11 The process allows for focusing of reviews where
12 necessary and it also provides for providing the current
13 licensing basis where, in support -- in the application we need
14 to support the analysis and the resulting conditions and that's
15 the extent to which we feel the current licensing basis should
16 need to be provided.

17 Again, we don't feel that it's necessary to submit
18 the current licensing basis nor confirm its accuracy and
19 completeness. Current programs to track or to continue to make
20 sure that we can identify the current licensing basis and keep
21 it implemented at the plant and don't undo something that we've
22 told the NRC that we're going to do, are in place in our
23 current licensee's programs for updating the FSAR, commitment
24 tracking, a number of things that we're doing, and the NRC has
25 access through its oversight programs to review these.

1 We think that provides an adequate basis for the
2 continued operation in the renewal period.

3 The backfit should be maintained during the
4 rulemaking process as well as during the review of our plant-
5 specific application. I think that moreso, we seek not to say
6 that changes shouldn't be made but that the backfit provides a
7 disciplined process which we can all look at and review and see
8 if changes are warranted.

9 Severe accident resolution is proceeding. It's
10 proceeding at Northern States Power. We're well along on our
11 IPE evaluation and participating in the NUMARC efforts in that
12 area too and we do not think it needs to be linked to the
13 renewal process. I think that some of the things I heard
14 through the technical sessions and things is that there's a
15 tendency to want to get every issue that's open on the books
16 today, closed, and make that if we can part of the license
17 renewal process.

18 I don't think that we want to say that there are no
19 "today" issues that shouldn't be put in there because there are
20 some that are very important and will impact the renewal
21 period. However, I think that we should be careful in
22 including issues that are really "today" issues in trying to
23 put them into the license renewal area. It's going to be
24 difficult enough to get through the license renewal process
25 without trying to solve all of the issues that we have on the

1 books today. We should attempt to keep them separate and let
2 the normal processes take their own course.

3 The three other quick points that I'd like to hit
4 because I at times wanted to stand up and respond to questions
5 but I didn't, so I'll take my opportunity now. I think the
6 lead plants and it's Northern States Power's intention, is to
7 utilize the industry reports to their fullest extent by
8 reference and there was some question as to the Section 6 items
9 which are basically those items which could not be
10 dispositioned on a generic basis, those age-related degradation
11 mechanisms and what the standing of those were.

12 We intend to address in our application those
13 findings which could not -- those items of age-related
14 degradation which could not be resolved generically and we will
15 address them in our application on a plant-specific basis when
16 we get into those components.

17 Another item that I guess I'd like to address is the
18 current codes and what the current code activities are in
19 attempting to address license renewal. As a NUMARC working
20 group member, I am also sitting on the Board of Nuclear Codes
21 and Standards Steering Committee on plant life extension. I'm
22 aware of the activities that they've got going to try to direct
23 the code-related activities and improving the codes for license
24 renewal and to provide information to those, that body, to
25 guide their activities and what they need to go address, we

1 have provided the follow on action item list from the pilot
2 study and I know that as well as the inputs we're getting as we
3 learn things from the industry reports is going to that and I
4 think we should continue on with that process and that's a very
5 important process and let the code groups address them as they
6 see appropriate.

7 Finally, I'd like to address schedule. I guess we
8 are in complete agreement with Yankee Atomic that we would like
9 to have the rulemaking issued in May of 1991 so that when we
10 submit our application, that it will actually be a
11 demonstration of that rulemaking and that we agree that we'd
12 like to also have the generic environmental process completed
13 in a time such that when we get our SER issued, that we can
14 close out some of those issues and I'd like to urge the staff
15 to work towards that schedule if it's at all possible.

16 That concludes my remarks.

17 MR. SPEIS: Thank you, Mr. Pickens.

18 Bill Rasin of NUMARC now will give us his summary and
19 his impressions of what happened here.

20 MR. RASIN: Thanks, Themis.

21 I'd like to make a few summary comments on behalf of
22 NUMARC and the industry.

23 I think this has been a very good workshop. I was
24 very happy to see the staff summary of the sessions, because it
25 shows me that they really have listened and, I think, captured

1 the comments that we had to bring here to this workshop.

2 I do want to make sure I have one opportunity here,
3 just so that there's no doubt in your mind that we think
4 there's some inconsistency between the philosophy and the rule,
5 and I wanted to make sure we got that point across.

6 I will remind Frank, thought, that he did ask that
7 question the first day, and I don't think we'll be accused of
8 be nonresponsive to that question, anyway.

9 I'm not going to review all of the points and issued
10 that have been made. I think the previous speakers just did a
11 pretty good job of that. I would like to hit a few high points
12 with a little bit of discussion as to what's behind our
13 thinking in some of those points.

14 We had the question of consistency in the application
15 of the current licensing basis and what information should be
16 supplied to respond to the rule and demonstrate the applicable
17 age-related degradation has been taken into account for
18 continued plant operation.

19 There was the comment made that the current licensing
20 basis applicability could be dealt with by making a generic
21 finding in the statement of considerations with the rule, and I
22 believe that's one that merits some consideration and may be a
23 way to resolve some of the difficulties and differences.

24 Our real dilemma here, I think, is that we need to
25 see the staff lay down the standards for the findings that need

1 to be made, and then we need to make sure that the information
2 provided is fully sufficient to make those findings. To supply
3 less information than that certainly is not in the interests of
4 the industry. To provide more information than that, however,
5 I think not only wastes resources in the industry, but I think
6 will cause an inefficient use of resources in the staff, and we
7 don't think either one of us can really afford, in this day and
8 age, to use our resources inefficiently.

9 We made some comments and you made some comments, and
10 I think, from your responses, that you will do some thinking in
11 the staff when you go back, and I assure you, we'll do some
12 more thinking in the industry over what you said and make sure
13 that our position really is consistent and that we are looking
14 to provide adequate information for you to make your findings.

15 Let me say just a word about the environmental
16 effects area. I have a lot of sympathy with the staff over
17 coming to grips with this area, because, I'll tell you, it's
18 one that has driven us crazy. Every time we think we
19 understand it and have a handle on it, we get another opinion
20 that sends us around the loop again.

21 I believe that we have stated the best position that
22 we can bring to fore at the time, and that's that we believe
23 that the schedule for the rule to be put in place in 1991,
24 consistent with the needs of the lead plant, is a major
25 necessity. To that end, an environmental assessment that

1 serves that rulemaking purpose should be done, and I think, as
2 acknowledged by the staff, could be done on that timeframe.

3 Now, the generic environmental assessment to resolve
4 generic issues, we think, also is a worthy goal. It's one that
5 certainly will save the staff resources, but clearly can save
6 the industry resources, as well.

7 The staff indicated a schedule for 1992 based on the
8 work necessary for that endeavor. We believe 1992 is probably
9 an acceptable date for that. Even though it will be behind the
10 rulemaking of part 50, it should have the information available
11 before decisions have to be made on the lead plants and
12 certainly before any other decisions have to be made.

13 So, that schedule, if the resources are available for
14 that one, we think probably they both could be accomplished
15 with about the same resources by that difference in schedule,
16 and I will reiterate NUMARC's offer to the staff that, seeing
17 the advantage in this process, we do stand ready to work with
18 the staff. The work is the staff's; the assessment is the
19 staff's. However, if we can provide input from the industry in
20 the form of data or analysis that can serve as input to the
21 staff's work, we will certainly consider doing so.

22 The backfit rule has been covered, and I think it
23 comes down to the point that Terry Pickens brought up. It's
24 simply discipline in the process.

25 We know, from long history in this business, that all

1 of us have our favorite issues, and I don't mean this to be
2 criticism of the staff, because I will say that the industry,
3 the utilities, our vendors, and certainly our consultants, are
4 just as guilty of finding the latest hot topic to latch on
5 their favorite issue to.

6 We have worked very, very hard, in our process in the
7 industry in our NUPLEX effort, not to let that happen on our
8 side, and we've had a lot of criticism and disagreement, but we
9 stuck fast to saying let's deal with this issue. It's hard
10 enough, and the other issues will be dealt with in their proper
11 forum.

12 Our concern with the backfit rule is simply that we
13 have that same disciplined approach on the part of the staff,
14 because to do otherwise will serve neither one of us.

15 Now, with regard to severe accidents, again it's been
16 a consistent position we don't like that in the rule. I must
17 tell you, I'm a little bit tempted to say yes, go ahead and put
18 it in the rule, because I assume that would also hold you, NRC
19 staff, necessary to complete your reviews by that rule.

20 However, I'm not so bold -- not that I don't have confidence in
21 you, but I think that that adds an unnecessary degree of
22 difficulty and complication. I will tell you that we are
23 committed to resolving the severe accident issue and having the
24 industry respond fully to the Commission's policy statement,
25 and we'll do everything we can to do so.

1 PRAs came up in Section 4. There was a very lively
2 discussion on the application of PRAs and some the very new
3 techniques on aging mechanisms. In that discussion -- that was
4 one of the more exciting discussions I viewed in the workshop -
5 - there was a very interesting technical disagreement between
6 three experts for whom I have a good bit of respect, but as
7 soon as I get disagreement between experts for whom I have a
8 great deal of respect, that tells me that technology is not
9 ready for application on a general basis.

10 I think that we should PRA as input where we have it
11 and for what it can tell us, but we are not ready to have PRA
12 be a requirement, and I don't think that the NRC is ready to
13 use PRA to make format licensing decisions.

14 With regard to having PRAs in place only for purposes
15 of the environmental aspects, I would submit that that's an
16 awfully expensive way to do something that we have done pretty
17 successfully other ways in the past.

18 There was some concern, in Session 5, that discussion
19 of maintenance took place, and there was not an adequate
20 response from the industry on those issues, although in the
21 slide that was put up on that, I think the right idea was
22 captured.

23 However, let me take about 30 seconds and answer the
24 first 9 questions, at least, on the maintenance, surveillance,
25 and testing.

1 We in the industry and you in the staff are putting a
2 lot of effort into the maintenance question. I do not think
3 that we have hidden our feelings too deeply that we in the
4 industry would rather not have a maintenance rule. However,
5 it's our burden right now to convince not you, the staff, but
6 the Commissioners that, in fact, we are doing enough in the
7 industry so that a rule is not necessary.

8 I don't think there is any misunderstanding on the part of
9 the Commissioners or the staff that we hold that position
10 because we're against maintenance or don't realize
11 maintenance's fully importance to the safety of operation of
12 nuclear power plants, but let me say, again, that's its own
13 issue. It is receiving tremendous attention in the industry,
14 and it should be solved on its own merits.

15 If it comes out that the wisdom of the Commission is
16 what there should be a maintenance rule, it will become part of
17 the current licensing basis, and I'm sure we'll take full
18 advantage of it for the purposes of license renewal.

19 If, on the other hand, it's concluded by those
20 Commissioners that it is not necessary, then I think that
21 decision should stand and we should not try to use this other
22 process to put in place things that were not gained through
23 that very extensive dialoguing process.

24 To that regard, I think that is the answer for about
25 at least the first nine questions, I don't think they need to

1 be answered one by one.

2 Industry, through the NUMARC NUPLEX process, does
3 have in place, still, a significant amount of work. We
4 promised the staff 10 industry reports. These reports are
5 being sponsored by DOE and EPRI, and we are committed to
6 getting those reports out on schedule. We have modified that
7 schedule with what we think is a schedule consistent with your
8 rulemaking schedule, and we'll do everything we can to make
9 sure that those reports get out on time and are submitted to
10 you, and we'll also look forward to your questions and comments
11 and will answer those as expeditiously as we can.

12 Finally, let me end up by saying that I, too, thought
13 this was an excellent workshop. I did have some comments from
14 a few people, particularly some of the consultants and lawyers,
15 that they missed the entertainment of industry and staff
16 yelling at one another and calling one another incompetent, but
17 I have to tell you, I didn't miss that. I think this was an
18 excellent example of a way that we can proceed forward, working
19 fully in public, but on a very professional level, to state our
20 views and trade our views and then act on them afterwards.

21 So, I thank you very much for this opportunity, and I
22 hope this is more indicative of the way we do business in the
23 IPU workshop in the future.

24 Thank you.

25 MR. SPEIS: Thank you, Bill.

1 Well, according to the schedule, we're right on time.
2 So, I would like to offer you some options.

3 There is coffee available, but possibly, the coffee
4 can wait, unless you think this discussion can go on much
5 longer. Maybe we can go on for the next 5 or 10 minutes and
6 see if there are any questions or some things that have been
7 somehow missed from the presentations or some thoughts that
8 were generated as the speakers went through their summaries and
9 see how we go from here.

10 So, let me open the floor to any questions or any
11 comments.

12 [No response.]

13 MR. SPEIS: No takers, huh?

14 Well, I guess we have done our work.

15 MR. GILLESPIE: We need to leave on a lighter note.

16 I would like to apologize for using the word
17 "initiator" in the first session. I will never do that again.

18 MR. SPEIS: Well, if there are no questions and if
19 there are no comments, that means that we have done our work
20 properly and and efficiently and effectively, and I would like
21 to take this opportunity to thank all of you for participating
22 in this very effective and educational seminar workshop.
23 Especially, I would like to thank the lead plant participants -
24 - Yankee and Northern States Power and NUMARC.

25 I would like, also, to thank Donna Matson for

1 organizing this workshop and doing it so effectively and having
2 coffee on time and water in front of the speakers and so and so
3 forth.

4 I wish you good trips back home, and hopefully, we
5 won't have to see you before the proposed rule.

6 Thank you.

7 [A, lause.]

8 [Whereupon, at 2:50 p.m., the meeting was adjourned.]

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REPORTER'S CERTIFICATE

This is to certify that the attached proceedings before the United States Nuclear Regulatory Commission

in the matter of:

NAME OF PROCEEDING: Summary of Concurrent Sessions
Public Workshop

DOCKET NUMBER:

PLACE OF PROCEEDING: Reston, VA

were held as herein appears, and that this is the original transcript thereof for the file of the United States Nuclear Regulatory Commission taken by me and thereafter reduced to typewriting by me or under the direction of the court reporting company, and that the transcript is a true and accurate record of the foregoing proceedings.



Kevin Mahoney

Official Reporter
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