



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DETROIT EDISON COMPANY  
WOLVERINE POWER SUPPLY COOPERATIVE, INCORPORATED

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 43  
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Detroit Edison Company (the licensee) dated November 14, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 43, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Albert W. McAggry for*  
John O. Thoma, Acting Director  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 14, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 43

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change.

REMOVE

3/4 3-16

3/4 3-17

INSERT

3/4 3-16

3/4 3-17

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
2. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>		
a. $\Delta$ Flow - High	$\leq 55.1$ gpm	$\leq 63.4$ gpm
b. Heat Exchanger/Pump/High Energy Piping Area Temperature - High	$\leq 175^{\circ}\text{F}^{**}$	$\leq 183^{\circ}\text{F}^{**}$
c. Heat Exchanger/Pump/Phase Separator Area Ventilation $\Delta$ Temperature - High	$\leq 50^{\circ}\text{F}^{**}$	$\leq 53^{\circ}\text{F}^{**}$
d. SLCS Initiation	NA	NA
e. Reactor Vessel Low Water Level - Level 2	$\geq 110.8$ inches*	$\geq 103.8$ inches
f. Deleted		
g. Manual Initiation	NA	NA
3. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>		
a. RCIC Steam Line Flow - High		
1. Differential Pressure	$\leq 87.0$ inches H <sub>2</sub> O	$\leq 95.0$ inches H <sub>2</sub> O
2. Time Delay	3 seconds	3 $\pm$ 2 seconds
b. RCIC Steam Supply Pressure - Low	$\geq 62$ psig	$\geq 53$ psig
c. RCIC Turbine Exhaust Diaphragm Pressure - High	$\leq 10$ psig	$\leq 20$ psig
d. RCIC Equipment Room Temperature - High	$\leq 150^{\circ}\text{F}^{**}$	$\leq 160^{\circ}\text{F}^{**}$
e. Manual Initiation	NA	NA

TABLE 3.3.2-2 (Continued)  
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
4. <u>HIGH PRESSURE COOLANT INJECTION SYSTEM ISOLATION</u>		
a. HPCI Steam Line Flow - High		
1. Differential Pressure	< 395.0 inches H <sub>2</sub> O	< 410.0 inches H <sub>2</sub> O
2. Time Delay	3 seconds	3±2 seconds
b. HPCI Steam Supply Pressure - Low	≥ 100 psig	≥ 90 psig
c. HPCI Turbine Exhaust Diaphragm Pressure - High	≤ 10 psig	≤ 20 psig
d. HPCI Equipment Room Temperature - High	≤ 150°F**	≤ 162°F**
e. Manual Initiation	NA	NA
5. <u>RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION</u>		
a. Reactor Vessel Low Water Level - Level 3	≥ 173.4 inches*	≥ 171.9 inches
b. Reactor Vessel (Shutdown Cooling Cut-in Permissive Interlock) Pressure - High	≤ 89.5 psig***	≤ 95.5 psig***
c. Manual Initiation	NA	NA

\*Above TAF. See Bases Figure B 3/4 3-1.

\*\*Initial setpoint. Final setpoint to be determined during startup test program. Any required change to this setpoint shall be submitted to the Commission within 90 days of test completion.

\*\*\*Represents steam dome pressure; actual trip setpoint is corrected for cold water head with reactor vessel flooded.