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John C. Brons Executive Vice President Nuclear Generation

November 17, 1989 JPN-89-077

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Mail Station P1-137 Washington, D.C. 20555

## SUBJECT: James A. FitzPatrick Nuclear Power Plant Docket No. 50-333 Containment Isolation Valve Replacement Schedule

Reference: NYPA letter, J. C. Brons to the NRC, "Request for Exemption from Containment Integrated Leak Rate Test - Retest Schedule," dated April 8, 1989 (JPN-88-012).

Dear Sir:

Attachment II to the referenced letter provided a schedule for the replacement of 33 containment isolation valves (CIV). Twenty one CIVs were scheduled for replacement during the 1988 refueling outage and the remainder during the 1990 refueling outage. All of the valves scheduled for replacement during 1988 were replaced on schedule. The replacement of four valves scheduled for the 1990 outage had been accelerated, and these valves were replaced during the 1989 maintenance outage. One of the remaining eight valves cannot be replaced during the 1990 outage as had been originally scheduled.

The Authority is having difficulty procuring a replacement for valve 20AOV-95 from the original manufacturer. Valve 20AOV-95 is a non-lubricated air operated plug valve, and the original manufacturer is the only supplier of this type of valve with a qualified Quality Assurance program in accordance with 10 CFR 50 Appendix B. The Authority evaluated replacing this valve with a different style valve, but this was deemed infeasible due to space limitations. Depending on the procurement problems, the Authority will either upgrade valve 20AOV-95 or replace this valve no later than the 1991 refueling outage.

Valve 20AOV-95 is the outboard CIV on the equipment drain sump to the radwaste system. The inboard CIV on this penetration, 20MOV-94, will be replaced during the 1990 refueling outage. This alone will effect a significant improvement in the penetration's long term integrated leakage characteristics. The existing outboard CIV 20AOV-95 will be Type C local leak rate tested in accordance with 10 CFR 50 Appendix J and repaired as necessary to assure its leak tightness prior to the startup following the refueling outage.

Should you or your staff have any questions regarding this matter, please contact S. M. Toth of my staff.

Very truly yours,

000 For John C. Brons

Executive Vice President Nuclear Generation

cc: Regional Administrator U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406

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