

November 17, 1989

RANCHO SECO UNIT 1  
TECHNICAL SPECIFICATIONS

Limiting Conditions for Operation

- 3.6.7 The Reactor Building purge valves, SFV 53503, SFV 53504, SFV 53604, and SFV 53605, shall be closed with their respective breakers de-energized, except during cold shutdown or refueling. Valves SFV 53503 and SFV 53604 shall be verified to be in the above condition at least monthly. The breakers/disconnects on valves SFV 53504 and SFV 53605 shall be verified to be de-energized at least monthly.
- 3.6.8 The Reactor Building purge valves and Reactor Building pressure equalization valves shall isolate on high containment radiation level. See Table 3.5.1-1 for operability requirements.

Bases

The reactor coolant system conditions of cold shutdown assure that no steam will be formed and hence no pressure buildup in the containment if the reactor coolant system ruptures.

The selected shutdown conditions are based on the type of activities that are being carried out and will preclude criticality in any occurrence.

The Reactor Building is designed for an internal pressure of 59 psig and an external pressure 2.0 psi greater than the internal pressure. The design external pressure corresponds to the differential pressure that could be developed if the building is sealed with an internal temperature of 120°F with a barometric pressure of 29.0 inches of Hg and the building is subsequently cooled to an internal temperature of 80°F with a concurrent rise in barometric pressure to 31.0 inches of Hg.

When containment integrity is established, the limits of 10 CFR 100 will not be exceeded should the maximum hypothetical accident occur.

The OPERABILITY of the containment isolation ensures that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere by pressurization of the containment. Containment isolation within the time limits specified ensures that the release of radioactive material to the environment will be consistent with the assumptions used in the analyses for LOCA.

Specifications 3.6.7 and 3.6.8 are in response to NUREG 0737, Item II.E.4.2.

REFERENCES

- (1) USAR, section 5

Amendment No. 27, 49, 87, 112, 113

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