

CONCLUSIONS

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

| | |
|---|---|
| 0 | 1 |
|---|---|

REPORT SOURCE

| | | | | | | | | | | | | | | | | | | | | | | | | |
|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|
| L | 6 | 0 | 5 | 0 | - | 0 | 2 | 9 | 3 | 7 | 0 | 9 | 1 | 1 | 8 | 0 | 8 | 0 | 9 | 1 | 9 | 8 | 0 | 9 |
|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|---|

60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 Management review concluded that station operation and maintenance on the containment

0 3 cooling system since the effective date of item No. 13 of Amendment N.42 of PNPS

0 4 Technical Specifications has been conducted in a manner less conservative than the

0 5 least conservative aspect of the limiting condition for operation established in

0 6 the Technical Specifications.

0 7

0 8

| | | | | | | | | | | | | | | | | | | | | | | |
|----------------------|----|---------------|----|-----------------------|----|-----------------|----|----------------|----|----------------------|----|------------------|----|----------------------|----|------------------------|----|----|----|----|----|----|
| 09 | | SYSTEM CODE | | CAUSE CODE | | CAUSE SUBCODE | | COMPONENT CODE | | | | | | COMP. SUBCODE | | VALVE SUBCODE | | | | | | |
| 0 | 9 | S | F | X | | Z | | Z | Z | Z | Z | Z | Z | Z | | Z | | | | | | |
| 7 | 8 | 9 | 10 | 11 | 12 | 12 | 13 | 13 | 14 | 15 | 16 | 17 | 18 | 19 | 20 | | | | | | | |
| LER/RO REPORT NUMBER | | EVENT YEAR | | SEQUENTIAL REPORT NO. | | OCCURRENCE CODE | | REPORT TYPE | | REVISION NO. | | | | | | | | | | | | |
| 17 | | 8 | 0 | — | | 0 | 5 | 0 | / | 0 | 1 | T | — | 0 | | | | | | | | |
| 21 | 22 | 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | | | | | | | | | | | |
| ACTION TAKEN | | FUTURE ACTION | | EFFECT ON PLANT | | SHUTDOWN METHOD | | HOURS | | ATTACHMENT SUBMITTED | | NPRD-4 FORM SUB. | | PRIME COMP. SUPPLIER | | COMPONENT MANUFACTURER | | | | | | |
| X | 18 | F | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | Y | 23 | N | 24 | Z | 25 | Z | 9 | 9 | 9 | 26 |
| 33 | 34 | 35 | 36 | 37 | 38 | 39 | 40 | 41 | 42 | 43 | 44 | 45 | 46 | 47 | 48 | 49 | 50 | 51 | 52 | 53 | 54 | 55 |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Root cause of this occurrence is sparse and ambiguous technical specifications for

1 1 operation with installed alternate shutdown panels.

1 2 _____

1 3 _____

1 4 _____

7 8 9 80

| FACILITY STATUS | | | N. POWER | | | OTHER STATUS | | | METHOD OF DISCOVERY | | DISCOVERY DESCRIPTION | | |
|-----------------|---|------|----------|----|----|--------------|------|----|---------------------|----|-----------------------|-------------------|----|
| 1 | 5 | (28) | 1 | 0 | 0 | (29) | N.A. | | | Z | (31) | Management Review | |
| 7 | 8 | 9 | 10 | 11 | 12 | 13 | 14 | 15 | 16 | 17 | 18 | 19 | 20 |

ACTIVITY CONTENT
RELEASED OF RELEASE

| | | | | | | | |
|---|---|---|----|----|----|------|--|
| 1 | 6 | 2 | 33 | 2 | 34 | N.A. | |
| 7 | 8 | 9 | 10 | 11 | 44 | | |

AMOUNT OF ACTIVITY (35)

LOCATION OF RELEASE (36)

| | |
|------|----|
| N.A. | |
| 45 | 80 |

| PERSONNEL EXPOSURES | | | | | | | | | | |
|---------------------|---|------|---|-------------|------|---|------|------|--|--|
| NUMBER | | TYPE | | DESCRIPTION | | | | | | |
| 1 | 7 | 0 | 0 | 0 | (37) | Z | (38) | N.A. | | |

| PERSONNEL INJURIES | | NUMBER | | DESCRIPTION | |
|--------------------|---|--------|---|-------------|------|
| 1 | 8 | 0 | 0 | 0 | N.A. |

| | | LOSS OF OR DAMAGE TO FACILITY | | |
|---|---|-------------------------------|-------------|------|
| | | TYPE | DESCRIPTION | |
| 1 | 9 | Z | (42) N.A. | (43) |

| | | | | | | | | | | | | | | | | | |
|----|--|---|--|----|--|------|--|--------------|--|--|--|--|--|--|--|--|--|
| 20 | | N | | 44 | | N.A. | | NRC USE ONLY | | | | | | | | | |
|----|--|---|--|----|--|------|--|--------------|--|--|--|--|--|--|--|--|--|

PHONE: 617-746-7900

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

Attachment to LER 80-050/01T-0

Description

On September 10, 1980 at approximately 1700 hours a management review concluded that a Technical Specification violation had occurred. The violation occurred on September 7, 1980.

On September 7, 1980 while performing surveillance tests prior to removing 'A' diesel generator from service it was determined that RBCCW pump P202D was inoperable. Although RBCCW pump P202D was inoperable, the Watch Engineer permitted the diesel generator to be removed from service at that time because Tech. Specs. recognizes these are installed spares in each of the RBCCW loops.

However, the September 10, 1980 management review concluded that since alternate shutdown panels installed during the last refueling outage restricted remote control of the RBCCW pump P202D was one of those two, that 'A' diesel generator should not have been removed from service.

Additionally, Salt Service Water pump maintenance had been performed in August 1980. Similar analysis indicates that less than the required pumps were available during these maintenance activities and that the required compensatory actions were not initiated.

Cause & Corrective Action

The sparse and ambiguous language of the Technical Specifications is believed to be the root cause of these occurrences in that the Limiting Condition for Operation of the Safe Shutdown Panels is not clearly defined.

Upon identification of the September 10, 1980 situation a reactor shutdown was immediately initiated but terminated at approximately 1810 hours on the same day, following the return of the 'A' diesel generator to service. Additionally, a priority 'A' maintenance request was issued to repair RBCCW pump P202D which was returned to service on September 11, 1980.

A memo has been issued to station Watch Engineers appraising them of this situation and recommending action to preclude its recurrence.

To provide long term corrective action it is contemplated that following additional review of the existing equipment and their requirements that two action items will be proposed.

1. A design change to plant equipment and,
2. Technical Specification revision to clarify the LCO's for operation with installed alternate shutdown panels.