

BIG ROCK POINT

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: [REDACTED] 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

| | | | | | | | | |
|-----|-------|---------------|-------|----------------|-------|--------------|----|-----------|
| 0 1 | 7 8 9 | LICENSEE CODE | 14 15 | LICENSE NUMBER | 25 26 | LICENSE TYPE | 30 | 57 CAT 58 |
|-----|-------|---------------|-------|----------------|-------|--------------|----|-----------|

CONT'

| | | | | | | |
|-----|---------------|-------|--------------|------------------|----------------|----|
| 0 1 | REPORT SOURCE | LL 60 | 0 5 0 1 5 15 | 7 0 18 11 4 8 10 | 0 19 11 2 8 10 | 80 |
|-----|---------------|-------|--------------|------------------|----------------|----|

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

- 0 2 Following intermittent automatic test circuit alarms, the sensor Channel D of the Reactor Depressurizing System was removed from service to vent the reactor water level transmitter LT3183. All four RDS loops remained operable for this condition with 2/3 logic instead of 2/4. No hazard occurred and the equipment was restored to operable status within 4 hours per Tech Spec 11.3.1.5.3. Similar to RO-80-21. Reportability based on Tech Spec 6.9.2.b(2).

7 8 9

| SYSTEM CODE | CAUSE CODE | CAUSE SUBCODE | COMPONENT CODE | COMP. SUBCODE | VALVE SUBCODE | REVISION NO. | | |
|-------------------------|---------------|-----------------------|------------------|---------------|----------------------|------------------|----------------------|------------------------|
| 0 9 | S I H 11 | L X 12 | 2 2 2 2 2 2 2 14 | 2 15 | 2 16 | 0 1 | | |
| 7 | 8 | 9 10 | 11 12 | 13 14 | 15 16 | | | |
| 17 LER/RO REPORT NUMBER | EVENT YEAR | SEQUENTIAL REPORT NO. | OCCURRENCE CODE | REPORT TYPE | | | | |
| 21 | 8 0 | 0 2 4 | 0 1 3 | L | | | | |
| 22 | 23 | 24 25 | 26 | 27 | 28 | 29 | | |
| ACTION TAKEN | FUTURE ACTION | EFFECT ON PLANT | SHUT DOWN METHOD | HOURS | ATTACHMENT SUBMITTED | NPRO-4 FORM SUB. | PRIME COMP. SUPPLIER | COMPONENT MANUFACTURER |
| E 18 | X 19 | Z 20 | Z 21 | 0 10 0 10 | N 23 | N 24 | Z 25 | 2 9 1 9 1 9 25 |
| 33 | 34 | 35 | 36 | 37 | 38 | 39 | 40 | 41 |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

- 1 0 In addition to minor conservative calibration shift occurring in the reactor water level transmitter, the setpoint for reactor low level is unduly conservative. A Technical Specification change request was submitted on 8/25/80 to provide greater margin in the trip setpoint.

7 8 9

| FACILITY STATUS | % POWER | OTHER STATUS | METHOD OF DISCOVERY | DISCOVERY DESCRIPTION | LOCATION OF RELEASE |
|-------------------------------|---------------------|--------------------|---------------------|------------------------------------|---------------------|
| 1 5 | E 28 | 0 1 6 5 23 | N/A | A 31 Operator observation of alarm | 36 |
| 7 | 8 9 | 10 12 13 | 44 | 45 46 | 80 |
| ACTIVITY CONTENT | RELEASED OF RELEASE | AMOUNT OF ACTIVITY | 35 | | |
| 1 6 | 7 33 | 7 24 | NA | NA | 43 |
| 7 | 8 9 | 10 11 | 64 | | 80 |
| PERSONNEL EXPOSURES | NUMBER | TYPE | DESCRIPTION | | |
| 1 7 | 0 0 0 37 | Z 38 | NA | | 80 |
| 7 | 8 9 | 11 12 13 | 44 | | |
| PERSONNEL INJURIES | NUMBER | DESCRIPTION | 41 | | |
| 1 8 | 0 0 0 40 | NA | | | 80 |
| 7 | 8 9 | 11 12 | 42 | | |
| LOSS OF OR DAMAGE TO FACILITY | TYPE | DESCRIPTION | 43 | | |
| 1 9 | 7 42 | NA | | | 80 |
| 7 | 8 9 | 10 | | | |
| PUBLICITY ISSUED | DESCRIPTION | 45 | | | |
| 2 0 | N 44 | NA | | | 80 |
| 7 | 8 9 | 10 | | | |

NRC USE ONLY

[REDACTED]

68 69 80

ORIGINAL
POOR

8009190/01