



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report No. 050-369/80-15

Licensee: Duke Power Company
422 South Church Street
Charlotte, NC 28242

Facility Name: McGuire Unit #1

Docket No. 050-369

License No. CPPR

Inspection at McGuire site on Lake Norman

Inspector:

R. D. Martin for
T. J. Donat

7/28/80

Date Signed

Approved by:

R. D. Martin
R. D. Martin, Section Chief, RONS Branch

7/28/80

Date Signed

SUMMARY

Inspection Dates: June 9 and 10, 1980; June 17 and 18, 1980

Areas Inspected

This routine unannounced inspection involved 36 inspector-hours on site in the areas of preparation for hot functional testing, preoperational test witnessing, review of licensee's responses to bulletins and circulars, review of previous inspection findings, and participation in the Emergency Planning review team audit.

Results

In the areas inspected, no items of noncompliance or deviations were identified.

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DETAILS

1. Persons Contacted

Licensee Employees

- *M. D. McIntosh, Plant Manager
- *E. Estep, Associate Licensing Engineer
- *G. Gage, Operation Superintendent
- *D. Lampke, Assistant Licensing Engineer
- L. Weaver, Performance Engineer
- M. Pacetti, Test Engineer
- M. Kitlan, Associate Test Engineer

Other licensee employees contacted involved technicians, operators, security force members and office personnel.

Other Organizations

- E. M. McP. Office of Nuclear Reactor Regulation, Emergency Response Plan Review
- J. Sears, Office of Nuclear Reactor Regulation, Emergency Response Plan Review Team Leader

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on June 18, 1980, with those persons indicated in Paragraph 1 above.

3. Licensee Action on Previous Inspection Findings

Closed - Unresolved Item 78-10-01 concerning allegations that Duke Power Company had performed inadequate and incorrect or misleading design calculations for the dropped spent fuel cask accident. The matter has been previously resolved and documented in IE report 50-369/78-20, 50-370/80-15, dated August 7, 1978.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Evaluation of Licensee's Emergency Response Program

The inspector accompanied the joint NRR/IE Emergency Response Preparedness review team on its tour of the Unit 1 portions of the control room, the technical support center, the operational support center, the shift supervisor's office, the primary coolant sample room, the counting room, the meteorology tower, and the personnel decontamination and first aid rooms.

Also, the following near site facilities were inspected: the environmental monitoring station, the proposed location for the emergency operation center (crisis management center) and the training center.

The inspector participated in the review team's meeting with licensee emergency planning representatives in order to identify those areas of the McGuire Emergency Response plan which needed additional information or clarification. On the evening of June 18, 1980, the inspector participated as a member of the emergency response preparedness review team at the Public Meeting on the Emergency Planning for the McGuire Nuclear Station held at the McGuire Training Center.

The inspector had no comments on the status of the licensee's Emergency Preparedness program or its implementation at the McGuire Nuclear Station. The conclusion and recommendations of the review team will be provided in a separate report.

6. Review of Hot Functional Test Program

The inspector reviewed the status of preoperational test procedures concerned with the second hot functional test sequence. This included verification that all necessary procedures had been approved for use by the station's Safety Review Committee (SRC) and that the necessary initial conditions had been signed and dated by cognizant licensee personnel.

The inspector did witness the performance of TP/1/A/1150/16A Annulus Ventilation Test. The test procedure was reviewed to insure that all prerequisites had been satisfied. The inspector discussed the test with the test engineer and observed the conduct of the vacuum bleed down rate portion of the test. The inspector observed the licensee's inspection of the door seals and his evaluation of the test data. The inspector's findings were clear in this area.

7. Review of Status of Licensee Response to IE Bulletins and Circulars

Closed - IE Bulletin 79-15 on Deep Draft Pumps - The licensee responded August 30, 1979, that no deep draft pumps were installed in safety related systems at the McGuire Nuclear Steam Plant. The inspector verified the information provided and this bulletin is closed.

Closed - IE Bulletin 79-16 on Vital Access Controls - The licensee, in their response dated June 18, 1980, stated that the subject of the adequacy of vital access controls had been addressed in their Site Security Plan which was under review by NRC's Office of Nuclear Material Safety and Safeguards. The bulletin is considered closed because the subject is presently undergoing review by a separate NRC organization.

Closed - IE Bulletin 79-17 on Pipe Cracks in Stagnant Borated Water Systems at BWR Plants - Not applicable to McGuire Nuclear Steam Plant which is a PWR facility.

Closed - IE Bulletin 79-24 on Frozen Lines in Safety Systems - The inspector reviewed the licensee's response dated October 31, 1979, in which they committed to insulate and heat trace the vent, recirculation, and level detection piping associated with the Refueling Water Storage Tank. The inspector reviewed the installation drawings for the modifications, and visually sited the affected piping. Heat trace wires were visible on all sections of the piping except for a six foot portion of the recirculation piping. This was identified to the licensee, who had it corrected. The bulletin is considered closed.

Closed - IE Bulletin 80-11 - Masonry Wall Design - This bulletin was not issued for licensee action at the McGuire Nuclear plant. The review of the licensee's submittals in response to IE Bulletins 79-02 and 79-14 will also cover the application of supports to masonry surfaces.

Closed - IE Bulletin 80-12 - Insuring Decay Heat Removal Capability - This bulletin was also issued only for facilities with an operating license and is not applicable to McGuire. However, the technical specification changes necessary to assure diverse redundant means of decay heat removal will be incorporated prior to issuance of an operating license on Unit 1.

The inspector verified that the following IE circulars had been received at the facility and routed to the appropriate management representative for action:

- 79-13 Replacement of Diesel Fire Pump Starting Contactors
- 79-17 Contact Problem in SB-12 Switches on G. E. company Metal Clad Circuit Breaker
- 79-18 Proper Installation of Target Rock Safety-Relief Valves
- 79-19 Loose Locking Devices on Ingersoll-Rand Pumps
- 79-20 Failure of GTE Sylvania Relays
- 79-22 Stroke Times for Power Operated Relief Valves
- 79-23 Failure of Motor Starters and Contactors to Operate
- 79-24 Proper Installation and Calibration of Core Spray Pipe Break Detection Equipment in BWR's
- 80-02 Nuclear Plant Staff Work Hours
- 80-07 Problems with the HPCI Turbine Oil System at BWR's

8. Followup on Previous Inspection Findings and Opened Items

- a. Closed - Open Item 79-06-03 which concerned the need for administrative guidance on the proper method of documenting valve checklists when more than one operator performed the verification. Station Directive 3.1.28, Rev. 3, dated 2/26/80 states in Paragraph 3 that when multiple operators each perform on a portion of a valve checklist that a master copy of the checklist will be maintained by the shift supervisor

and will be marked upon completion by the operators who will also attach their working copy of the valve checklist to the back of the master copy.

- b. Closed - Inspector Followup Item 79-13-01 on cleanup of the resin inadvertently added to the reactor coolant system. The inspector reviewed the licensee's actions including flushing the NC system and considers the item closed.
- c. Closed - Open Item 79-13-02 on adding the diesel generator battery, which provides excitation and control power to the diesel generator, to the plant's Technical Specifications. The "proof and review" copy of the Technical Specifications, dated June 20, 1980, has added surveillance requirement 4.8.1.1.4 concerning weekly checks of the diesel generator battery condition. The addition of these surveillance steps closes this item.
- d. Closed - Inspector Followup Item 80-03-04 concerned changing PT1/A/4151/02 to include measuring valve closure timing for PORV valves INC-32A, INC-34A, and INC-36A. The inspector reviewed Revision 1 to the periodic test dated June 16, 1980, which incorporated the measurement of the closure time.
- e. Closed - Inspector Followup Item 80-03-06 concerning whether the licensee evaluated the problems encountered with Cutler-Hammer type B26-MF relay decks for potential Part 21 reporting. The licensee has performed their evaluation and submitted a report to the commission in accordance with 10 CFR 50.55(e).