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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

August 12, 1980

Docket No. 50-293

Mr. G. Carl Andognini Boston Edison Company M/C NUCLEAR 800 Boylston Street Boston, Massachusetts 02199

Dear Mr. Andognini:

The enclosure is a request for additional information regarding BECo letter 76-11 of January 27, 1976. This letter proposed exemption to 10 CFR 50 Appendix J (Containment Leakage Testing).

Your response is requested within 45 days of the receipt of this letter. Any questions should be addressed to your project manager.

Sincerely,

Thomas M. Novak, Assistant Director for Operating Reactors Division of Licensing

Enclosure: Request for Additional Information

cc w/enclosure: See next page

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Mr. G. Carl Andognini Boston Edison Company

cc:

Mr. Paul J. McGuire Pilgrim Station Acting Manager Boston Edison Company RFD #1, Rocky Hill Road Plymouth, Massachusetts 02360

Henry Herrmann, Esquire Massachusetts Wildlife Federation 151 Tremont Street Boston, Massachusetts 02111

Plymouth Public Library North Street Plymouth, Massachusetts 02360

Resident Inspector c/o U. S. NRC P. O. Box 867 Plymouth, Massachusetts 02360 REQUEST FOR ADDITIONAL INFORMATION IMPLEMENTATION OF 10CFR50, APPENDIX J, CONTAINMENT LEAKAGE TESTING PILGRIM STATION UNIT 1

1.0 BACKGROUND

By a letter dated January 27, 1976, Boston Edison Company (BEC) requested certain exemptions from the requirements of 10CFR50, Appendix J, Containment Leakage Testing for Pilgrim Station Unit 1.

2.0 INFORMATION REQUIRED

2.1 FEEDWATER SYSTEM CHECK VALVES

BEC requested an exemption to test feedwater check values with water as a medium in lieu of air or nitrogen. This approach is acceptable provided the test is used to verify that the check value will remain water-covered throughout the post-accident period.

Provide the following information needed to complete our review of this request:

- The initial volume of water in the line at the start of the accident.
- The acceptance criteria for the hydrostatic test (e.g., leakage rate limit, pressure decay limit, etc.).
- 2.2 ISOLATION CHECK VALVES IN THE REACTOR WATER CLEAN-UP RETURN, REACTOR CORE ISOLATION COOLING (RCIC) PUMP DISCHARGE, HIGH PRESSURE COOLANT INJECTION (HPCI) PUMP DISCHARGE, CORE SPRAY TO REACTOR, AND RESIDUAL HEAT REMOVAL (RHR) VESSEL INJECTION LINES

BEC has stated that replacing these check values with air-testable check values was not considered justified since the purpose of these values is to limit reverse direction flow in case of a postulated upstream pipe break until downstream motor-operated isolation values are shut. The statement implies that testing of these values with water as a medium is at least possible and perhaps currently required.

Provide your proposal to verify that these values will limit reverse direction flow for a sufficient period of time to permit downstream isolation including:

 Indications available to the operator to identify a line which should be isolated by its downstream motor-operated isolation value.

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- The initial water volume in the line at the start of the accident.
- 3. The acceptance criteria for any required test.

Additionally, provide the following information needed to complete all aspects of this review:

- For each of the penetrations involved (9a,b; 10 a,b; 51a,b) identify the valves installed to perform a containment isolation function in accordance with General Design Criterion 55 (10CFR50, Appendix A).
- Indicate the location (inside or outside containment), type valve, and automatic features for each of the valves of paragraph 1 above.