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SACRAMENTO MUNICIPAL UTILITY DISTRICT 🖂 6201 S Street, Box 15830, Sacramento, California 95813; (916) 452-3211

RJR 80-379

August 6, 1980

Director (10) Office of Inspection & Enforcement U. S. Nuclear Regulatory Commission WAshington, D. C. 20555

Re: Operating Plant Status Report

Dear Sir:

Enclosed is the July Operating Plant Status Report for

Rancho Seco Unit One.

Respectfully submitted,

R. J. Rodriguez Manager, Nuclear Operations

jim

Eacl.

cs: Region V (1) Office of Management Information and Program Control (2) EPRI-NSAC (1)



## NARRATIVE SUMMARY OF PLANT OPERATIONS

7-1 to 7-23	Reactor at 96%.
<u>7-23</u> 0100	Reduced Reactor power to ${\sim}53\%$ due to condenser tube leak.
1739	Began increasing reactor power to 92% at 10 MWe/hr for 2-hour hold.
2108	Began increasing reactor to 96%.
2150	Reactor at 96%.
7-23 to 7-31	Reactor at 96%.

#### SUMMARY OF CHANGES MADE IN ACCORDANCE WITH 10 CFR 50.59(b)

The technical specification changes for cycle 4, shown in the Cycle 4 Reload Report, BAW-1560, show that the proposed boron concentration is sufficient to maintain k of 0.95 or less with all control rods removed from the core. This technical specification removes all restrictions on movement of the control rod assemblies during fuel shuffling. The FSAR statement is therefore not necessary and unrestricted movement of control rods will not increase the consequences of any accident previously evaluated in the FSAR.

### CYCLE 4 POWER DISTRIBUTION COMPARISON

In the District's letter to Mr. Robert W. Reid, dated February 27, 1980, we committed to perform Power Distribution Comparisons through Cycle 4 as a result of this being our first reload core ucilizing Lumped Burnable Poisons.

The "50 EFPD" data was obtained July 8, 1980, with the burnup at approximately 51.5 EFPD. Preliminary analysis of this data by B&W has determined the RMS value to be 0.0298, with the continuing requirement that it be less than 0.0731.

The next data is to be taken at near 100 EFPD, which is anticipated to occur in September, 1980.

#### FOLLOWUP ON REPORTABLE OCCURRENCE 80-15

During inspection of fuel discharged at the end of operating Cycle 3, one fuel assembly was found to have defective cladding on one fuel pin. That finding was reported in the referenced RO 80-15. Subsequent discussions with the NRC staff led to a request that a followup report be submitted which would relate the BOC-4 experience regarding fuel integrity. These data are the nominal values obtained at near full power:

	BOC-3	EOC-3	BOC-4 (thru 63 EFPD)
I-131	1.0 (10 <sup>-2</sup> )	1.5(10 <sup>-2</sup> )	5.0(10 <sup>-3</sup> ) µCi/cc
7-133	4.0(10 <sup>-2</sup> )	6.0(10 <sup>-2</sup> )	2.5(10 <sup>-2</sup> ) µCi/cc
	0.003	0.005	0.001 %FF

These data substantiate that the failed fuel pin had likely been so for some time, and that it was not an impediment to continued operation of the unit. It also shows that the failed pin was located and discharged. Failed fuel amounting to 0.001%FF is less than one equivalent fuel pin. This implies that, at this time, the core is effectively defect free.

This report closes this item.

# REFUELING INFORMATION REQUEST

1.	Name of Facility: Rancho Seco Unit 1
	Scheduled date for next refueling shutdown: July 1981
	Scheduled date for restart following refueling: September 1981
	Technical Specification change or other license amendment required:
	<ul> <li>a) Change to Rod In Power Level Curve (TS 3.5.2)</li> <li>b) Change to Core ance vs. Power Level Curve (TS 3.5.2)</li> <li>c) Tilt Limits (TS 3.5.2)</li> <li>d) Safety Equipment Testing (TS 3.3.3)</li> </ul>
5.	Scheduled date(s) for submitting proposed licensing action: May 1981
	Important licensing considerations associated with refueling: None
7.	Number of fuel assemblies:
	<ul> <li>a) In the core: <u>177</u></li> <li>b) In the Spent Fuel Pool: <u>164</u></li> </ul>
٤.	Present licensed spent fuel capacity: 579
9.	Projected date of the last refueling that can be discharged to the Spent Fuel Pool: 1787