

ATTACHMENT "B"

8008120

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(c) Corrective Action: Valve malfunctions disclosed by any Class C test shall be corrected and reported in the subsequent Class A test report.

Leaks which exceed the acceptance criteria of (b) shall be repaired and retested until the criteria are met. Repairs of lesser leaks are optional.

(d) Test Frequency: At least two Class C tests at approximately equally spaced intervals shall be performed during the interval between any scheduled Class A test.

5.2.1.4 Permissible Periods for Testing: The performance of Class A tests shall be limited to periods when the plant facility is nonoperational and secured in the shutdown condition under administrative control and safety procedures.

5.2.1.5 Report of Test Results: Class A tests shall be the subject of a summary technical report, which includes a schematic arrangement of the leakage measurement system, the instrumentation employed, the test procedure, test results in graphical form, and the analysis and interpretation of results. Summaries of Class B and C tests performed during the interval preceding the scheduled Class A test are to be included in the same report.

5.2.2 The reactor building isolation system will be tested for proper operation prior to every cold startup, but this test will not be required more often than at 30-day intervals.

5.2.3 The exterior surfaces of the LACBWR ventilation stack and the smoke stack of the conventional steam power generating station, Genoa 3, adjacent to the LACBWR plant shall be inspected for structural integrity at an interval no longer than 5 years following the initial construction inspection, and at subsequent intervals not longer than 5 years apart.

5.2.4 The reactor vessel shall be hydrostatically tested at 1400 psig after any of its gasketed joints have been opened and resealed. All hydrostatic tests shall be performed with the vessel at a temperature no lower than that specified in Sec. 4.2.2.4.

5.2.5 The forced circulation system controls and automatically-operated valves shall be tested for proper operation annually.

5.2.6 The shutdown condenser system control valves shall be tested at least quarterly to demonstrate their operability. The integrated system shall be tested for proper operation annually. In addition, the condenser tube bundle shall be pressurized to greater than 1250 psig and tested for leakage annually.

5.2.7 The high-pressure core spray system controls and remotely-operated valves shall be tested during cold shutdowns, but not more often than every 3 months, to demonstrate their operability. The integrated system shall be tested for proper operation annually.

5.2.8 The Alternate Core Spray System controls and remotely-operated valves shall be tested during cold shutdowns, but not more often than every 3 months, to demonstrate their operability. The integrated system shall be tested for proper operation annually, with the supply line to the reactor vessel isolated and the coolant flow directed through the system and back to the river. In addition, an annual test shall be performed to demonstrate that the check valves in the system are not stuck in a closed position.

5.2.9 The boron-injection system controls and the remotely-operated valves shall be tested for proper operation during cold shutdowns, but not more often than every 3 months.

(Next number is 5.2.12)

5.2.12 Each control rod scram time shall be measured prior to cold start-ups and the total scram time shall be demonstrated to be within the limit specified in Sec. 4.2.5.1. These scram time tests will not be required more often than at 30-day intervals, but at least semi-annually, unless the reactor vessel head has been removed or unless maintenance work has been performed on the control rod drives.

5.2.13 Each control rod drive mechanism shall be exercised by moving each partially or fully withdrawn control rod at least one-half inch in any one direction:

- a. At least once per 7 days during the initial 31 days of Cycle 5 operation, and
- b. At least once per 31 days thereafter.

5.2.14 Proper operation of both control solenoids for each of the hydraulic scram valves will be determined annually by visual observation of solenoid arm position following control rod scram.

5.2.15 Instrumentation shall be checked, tested and calibrated as indicated in the following chart.

ATTACHMENT "C"

INSERVICE INSPECTION PROGRAM - LACBWR CLASS I COMPONENTS

<u>ITEM NUMBER</u>	<u>EXAMINATION CATEGORY</u>	<u>COMPONENTS AND PARTS TO BE EXAMINED</u>	<u>METHOD</u>	<u>EXTENT AND FREQUENCY OF EXAMINATION</u>	<u>NOTES/REMARKS</u>
<u>REACTOR VESSEL AND CLOSURE HEAD</u>					
B1.1	B-A (1)	Longitudinal and circumferential shell welds. Meridional and circumferential head welds. Vessel-to-flange and head-to-flange circumferential welds	Volumetric	100%/10 Yrs. 100%/10 Yrs. 100%/10 Yrs.	Non Exist
B1.2	B-B (1)				
B1.4	B-D (1)	Primary nozzle-to-vessel welds and nozzle inside radiused section.	Volumetric	100%/10 Yrs.	
		a. Recirculation Outlet Nozzle No. 1			
		b. Recirculation Outlet Nozzle No. 2			
		c. Recirculation Outlet Nozzle No. 3			
		d. Recirculation Outlet Nozzle No. 4			
		e. Recirculation Inlet Nozzle No. 5			
		f. Recirculation Inlet Nozzle No. 6			
		g. Recirculation Inlet Nozzle No. 7			
		h. Recirculation Inlet Nozzle No. 8			

NOTE: (1) See relief request.

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INSERVICE INSPECTION PROGRAM - LACBWR CLASS I COMPONENTS

<u>ITEM NUMBER</u>	<u>EXAMINATION CATEGORY</u>	<u>COMPONENTS AND PARTS TO BE EXAMINED</u>	<u>METHOD</u>	<u>EXTENT AND FREQUENCY OF EXAMINATION</u>	<u>NOTES/REMARKS</u>
<u>REACTOR VESSEL AND CLOSURE HEAD - (Cont'd)</u>					
		i. Steam Nozzle N at 14°			
		j. Steam Nozzle S at 194°			
		k. Emergency Cooling Nozzle in Upper Shell at 315°			
		l. Emergency Cooling Nozzle in Upper Head Access Nozzle Flange			
		m. Upper Head Access Nozzle			
		n. Blow-Down Nozzle in Lower Head			
B1.5	B-E (1)	Vessel penetrations, including control rod drive and instrumentation penetrations	Visual	25%/10 Yrs.	
		a. Control Rod Drive Penetrations			
		b. Experimental Access Penetration 45°			
		c. Experimental Access Penetration at 165°			
		d. Experimental Access Penetration at 285°			
		e. Upper Liquid Level Penetration			
		f. Intermediate Liquid Level Penetration			
		g. Lower Liquid Level Penetration			

INSERVICE INSPECTION PROGRAM - LACBWR CLASS I COMPONENTS

<u>ITEM NUMBER</u>	<u>EXAMINATION CATEGORY</u>	<u>COMPONENTS AND PARTS TO BE EXAMINED</u>	<u>METHOD</u>	<u>EXTENT AND FREQUENCY OF EXAMINATION</u>	<u>NOTES/REMARKS</u>
		h. Primary Purification Penetration in Lower Head			
B1.6	B-F	Nozzle-to-safe end welds	Volumetric & Surface	100%/10 Yrs.	
		a. Emergency Cooling Nozzle in Upper Shell-to-Safe End			
		b. Blow-Down Nozzle in Lower Head-to-Safe End			
B1.7	B-G-1	Nuts	Surface or Volumetric & Visual	100%/10 Yrs.	
B1.8	B-G-1	Studs	Volumetric & Surface	100%/10 Yrs.	
B1.9	B-G-1	Ligaments between threaded stud holes	Volumetric	100%/10 Yrs.	
B1.10	B-G-1	Closure washers, bushings	Visual	100%/10 Yrs.	None Exist
B1.11	B-G-2	Pressure-retaining bolting	visual	100%/10 Yrs.	
		a. Upper Head Access Nozzle Flange Studs			
		b. Upper Head Access Nozzle Flange Nuts			
B1.12	B-H (1)	Integrally welded vessel supports	Volumetric	100%/10 Yrs.	
B1.13	B-1-1	Closure head cladding	Visual & Surface	6 Patches/10 Yrs.	
B1.14	B-1-1 (1)	Vessel Cladding	Visual	Each Refueling	
B1.15	B-N-1	Vessel interior	Visual	Each Refueling	

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INSERVICE INSPECTION PROGRAM - LACBWR CLASS I COMPONENTS

<u>ITEM NUMBER</u>	<u>EXAMINATION CATEGORY</u>	<u>COMPONENTS AND PARTS TO BE EXAMINED</u>	<u>METHOD</u>	<u>EXTENT AND FREQUENCY OF EXAMINATION</u>	<u>NOTES/REMARKS</u>
B1.16	B-N-2	Interior attachments and core support structures	Visual	100%/10 Yrs.	
B1.17	B-N-3	Core-support structures	Visual	NA	PWR's Only
B1.18	B-O	Control rod drive housings	Volumetric	100%/10 Yrs.	None Exist
B1.19	B-P	Exempted components	Visual (IWA-5000)	100%/10 Yrs.	
<u>HEAT EXCHANGERS</u>					
B3.1	B-B	Longitudinal and circumferential welds, including tube sheet-to-head or shell welds on the primary side.  a. Decay Heat Channel Head-to-Tube Sheet Weld	Volumetric	5%/10 Yrs.	
B3.2	B-D	Nozzle-to-head welds and nozzle inside radiused section on the primary side  a. Inlet Decay Heat Nozzle-to-Channel Head  b. Outlet Decay Heat Nozzle-to-Channel Head	Volumetric	100%/10 Yrs.	
B3.3	B-F	Nozzle-to-safe end welds	Volumetric & Surface	100%/10 Yrs.	
B3.5	B-G-1	Pressure-retaining bolts and studs	Volumetric & Surface	100%/10 Yrs.	None Exist
B3.6	B-G-1	Pressure-retaining bolting	Visual	100%/10 Yrs.	None Exist



INSERVICE INSPECTION PROGRAM - LACBWR CLASS I COMPONENTS

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B3.7	B-H	Integrally welded vessel supports	Volumetric	100%/10 Yrs.		None Exist
B3.8	B-1-2	Vessel Cladding a. Decay Heat Cooler	Visual	1 Patch/10 Yrs.		
B3.9	B-P	Exempted components	Visual	100%/10 Yrs.		
B3.10	B-G-2	Pressure-retaining bolting a. Decay Heat Cooler Head Studs, 56-03-001 b. Decay Heat Cooler Head Nuts, 56-03-001 c. Primary Purification Regenerative Cooler Head Studs d. Primary Purification Regenerative Cooler Head Bolts	Visual	100%/10 Yrs.		
B3.11	B-Q	Steam generator tubing	Volumetric	3%/10 Yrs.		None Exist
<u>PIPING PRESSURE BOUNDARY</u>						
B4.1	B-F (1)	Safe-end to piping welds and safe-end in branch piping welds a. 2-Inch Purification Branch b. 2-Inch Boss on Loop Isolation Valves c. 8-Inch Decay Heat Suction	Volumetric & Surface	100%/10 Yrs.		

INSERVICE INSPECTION PROGRAM - LACBWR CLASS I COMPONENTS

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		d. 6-Inch Decay Heat Discharge			
		e. Decay Heat Cooler Inlet and Outlet			
		f. 8-Inch Steam Lines			
		g. 10-Inch Steam Lines			
		h. 6-Inch Alternate Core Spray Line			
		i. 4-Inch Alternate Core Spray Line			
		j. 2½-Inch Boron Inject Line			
B4.2	B-G-1	Pressure-retaining bolts and studs in place	Volumetric		None Exist
B4.3	B-G-1	Pressure-retaining bolts and studs when removed	Volumetric & Surface		None Exist
B4.4	B-G-1	Pressure-retaining bolting	Visual		None Exist
B4.5	B-J (1)	Circumferential and longitudinal pipe welds	Volumetric	25%/10 Years	
B4.6	B-J	Branch pipe connection welds exceeding 6 inches in diameter	Volumetric	25%/10 Years	
B4.7	B-J	Branch pipe connection welds 6 inches in diameter and smaller	Surface	25%/10 Years	
B4.8	B-J	Socket welds	Surface	25%/10 Years	
B4.9	B-K-1 (1)	Integrally welded supports	Surface	25%/10 Years	

INSERVICE INSPECTION PROGRAM - LACBWR CLASS I COMPONENTS

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B4.10	B-K-2	Support components	Visual	100%/10 Yrs.	
B4.11	B-P	Exempted components	Visual	100%/10 Yrs.	
B4.12	B-G-2	Pressure-retaining bolting	Visual	100%/10 Yrs.	
		A. Purification filter cover			
<u>PUMP PRESSURE BOUNDARY</u>					
B5.2	B-G-1	Pressure-retaining bolts and studs when removed	Volumetric & Surface	100%/10 Yrs.	
		A. Forced Circulation Pumps Casing			
B5.3	B-G-1	Pressure-retaining bolting	Visual	100%/10 Yrs.	
B5.4	B-K-1	Integrally welded supports	Volumetric	25%/10 Yrs.	None Exist
B5.5	B-K-2	Support components	Visual	100%/10 Yrs.	
B5.6	B-L-1	Pump casing welds	Volumetric	100%/10 Yrs.	None Exist
B5.7	B-L-2	Pump casings	Visual	100%/10 Yrs.	
B5.8	B-P	Exempted components	Visual	100%/10 Yrs.	
B5.9	B-G-2	Pressure-retaining bolting	Visual	100%/10 Yrs.	
		A. Forced Circulation Pump Seal Cover			



INSERVICE INSPECTION PROGRAM - LACBWR CLASS I COMPONENTS

<u>ITEM NUMBER</u>	<u>EXAMINATION CATEGORY</u>	<u>COMPONENTS AND PARTS TO BE EXAMINED</u>	<u>METHOD</u>	<u>EXTENT AND FREQUENCY OF EXAMINATION</u>	<u>NOTES/REMARKS</u>
<u>VALVE PRESSURE BOUNDARY</u>					
B6.2	B-G-1	Pressure-retraining bolts and studs when removed  a. Forced Circulation Rotoport Upper Head	Volumetric & Surface	100%/10 Yrs.	
B6.3	B-G-1	Pressure-retaining bolting	Visual	100%/10 Yrs.	None Exist
B6.4	B-K-1	Integrally welded supports	Volumetric	25%/10 Yrs.	None Exist
B6.5	B-K-2	Support components	Visual	100%/10 Yrs.	None Exist
B6.6	B-M-1	Valve-body welds	Volumetric	100%/10 Yrs.	None Exist
B6.7	B-M-2	Valve Bodies	Visual	As Applicable (1 of a type)	(2)
B6.8	B-P	Exempted components	Visual	100%/10 Yrs.	
B6.9	B-G-2	Pressure-retaining bolting  a. Forced Circulation Rotoport Lower Head  b. Main Steam Relief valves in Mounting Flange  c. Decay Heat Pump Check Valve  d. Main Steam Rotoport Valve - Head  e. Feedwater Check Valve	Visual	100%/10 Yrs.	

NOTES: (2) See Letter, Madgett to Keppler, LAC-2786, dated October 8, 1974.

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RELIEF FOR CLASS I COMPONENTS  
REQUIRING ISI TO ASME XI

ITEM NO.	EXAMINATION CATEGORY TABLE IWB-2500	IDENTIFICATION OF WELD OR COMPONENT	INSPECTION REQUIRED BY IWB-2600	BASIS FOR REQUESTING RELIEF
B1.1	B-A	<u>REACTOR VESSEL LONGITUDINAL WELDS AND CIRCUMFERENTIAL WELDS IN CORE REGION</u>  LONGITUDINAL JOINT #12  CIRCUMFERENTIAL JOINT #13 LONGITUDINAL JOINT #14 CIRCUMFERENTIAL JOINT #15 LONGITUDINAL JOINT #16 CIRCUMFERENTIAL JOINT #17 LONGITUDINAL JOINT #18	VOLUMETRIC	A CONCRETE SHIELD WALL AROUND THE VESSEL IN THIS AREA PREVENTS ACCESS TO THE WELDS.  
B1.2	B-B	<u>REACTOR VESSEL LONGITUDINAL AND CIRCUMFERENTIAL WELDS NOT IN THE CORE REGION</u>  LONGITUDINAL JOINT #6  CIRCUMFERENTIAL JOINT #7 LONGITUDINAL JOINT #8 CIRCUMFERENTIAL JOINT #9 LONGITUDINAL JOINT #10 CIRCUMFERENTIAL JOINT #11 CIRCUMFERENTIAL JOINT #19 LONGITUDINAL JOINT #20 CIRCUMFERENTIAL JOINT #21	VOLUMETRIC	A CONCRETE BLOCK SHIELD WALL AROUND THE VESSEL IN THIS AREA PREVENTS ACCESS TO THE WELDS.  

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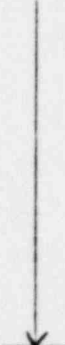
RELIEF FOR CLASS I COMPONENTS  
REQUIRING ISI TO ASME XI

ITEM NO.	EXAMINATION CATEGORY TABLE IWB-2500	IDENTIFICATION OF WELD OR COMPONENT	INSPECTION REQUIRED BY IWB-2600	BASIS FOR REQUESTING RELIEF
B1.4	B-D	<u>REACTOR VESSEL RECIRCULATION NOZZLES</u> OUTLET NOZZLES #1 AND INNER RADIUS OUTLET NOZZLE #2 AND INNER RADIUS INLET NOZZLE #5 AND INNER RADIUS INLET NOZZLE #6 AND INNER RADIUS	VOLUMETRIC	RADIATION LEVELS OF 1100 TO 1200 MR ON T <sub>1</sub> SURFACE WOULD CAUSE EXCESSIVE PERSONNEL EXPOSURES. THESE FOUR NOZZLES ARE SUBJECTED TO THE SAME THERMAL/HYDRAULIC CONDITIONS AS THE FOUR REACTOR VESSEL RECIRCULATION NOZZLES IN THE INSPECTION PROGRAM.
B1.4	B-D	BLOWDOWN NOZZLE IN LOWER HEAD AND INNER RADIUS	VOLUMETRIC	NO ACCESS DUE TO CRD NOZZLES. ALSO, THIS NOZZLE HAS BEEN CAPPED WHICH ELIMINATED THE DISSIMILAR METAL WELD.
B1.5	B-E	CONTROL ROD DRIVE PENETRATION TO REACTOR VESSEL	VISUAL (IWA-5000)	NO ACCESS BECAUSE THESE PARTIAL-PENETRATION WELDS ARE INSIDE THE REACTOR VESSEL.
B1.5	B-E	INTERMEDIATE LIQUID LEVEL PENETRATION TO REACTOR VESSEL	VISUAL (IWA-5000)	NO ACCESS BECAUSE THESE PARTIAL-PENETRATION WELDS ARE INSIDE THE REACTOR VESSEL.
B1.5	B-E	LOWER LIQUID LEVEL PENETRATION TO REACTOR VESSEL	VISUAL (IWA-5000)	NO ACCESS BECAUSE THESE PARTIAL-PENETRATION WELDS ARE INSIDE THE REACTOR VESSEL.
B1.5	B-E	PRIMARY PURIFICATION PENETRATION IN LOWER HEAD	VISUAL (IWA-5000)	NO ACCESS BECAUSE THESE PARTIAL-PENETRATION WELDS ARE INSIDE THE REACTOR VESSEL.
B4.5	B-J	INTERMEDIATE LIQUID LEVEL PENETRATION TO EXTENSION PIPE	VOLUMETRIC	NO ACCESS BECAUSE THE WELDS ARE INSIDE THE THERMAL SLEEVE WHICH PENETRATES THE BIOLOGICAL SHIELD WALL
B4.5	B-J	LOWER LIQUID LEVEL PENETRATION TO EXTENSION PIPE	VOLUMETRIC	NO ACCESS BECAUSE THE WELDS ARE INSIDE THE THERMAL SLEEVE WHICH PENETRATES THE BIOLOGICAL SHIELD WALL.

RELIEF FOR CLASS I COMPONENTS  
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ITEM NO.	EXAMINATION CATEGORY TABLE IWB-2500	IDENTIFICATION OF WELD OR COMPONENT	INSPECTION REQUIRED BY IWB-2600	BASIS FOR REQUESTING RELIEF
B4.5	B-E	PRIMARY PURIFICATION PENETRATION IN LOWER HEAD TO EXTENSION PIPE	VOLUMETRIC	NO ACCESS DUE TO REACTOR VESSEL SUPPORT RING.
B1.12	B-H	INTEGRALLY WELDED REACTOR VESSEL SUPPORTS	VOLUMETRIC	RELIEF REQUESTED TO USE ALTERNATE INSPECTION METHOD UTILIZING SURFACE NDE.
B1.14	B-I-1	REACTOR VESSEL CLADDING  <u>20" FORCED CIRCULATION SUCTION LOOP 1B</u>	VISUAL	NO ACCESS JUSTIFIED DUE TO HIGH RADIATION LEVELS.
B4.1	B-F	22 BC, 2" BRANCH CONNECTION  <u>2" FORCED CIRCULATION BYPASS LOAD 1A</u>	VOLUMETRIC	CONFIGURATION PRECLUDES VOLUMETRIC
B4.1	B-F	VALVE TO PIPE WELD #91A	VOLUMETRIC	CONFIGURATION PRECLUDES VOLUMETRIC
B4.1	B-F	PIPE TO VALVE WELD #90A  <u>2" FORCED CIRCULATION BYPASS LOOP 1B</u>	VOLUMETRIC	EXAMINATION. RELIEF REQUESTED TO DO SURFACE NDE INSPECTION.
B4.1	B-F	VALVE TO PIPE WELD #93A	VOLUMETRIC	CONFIGURATION PRECLUDES VOLUMETRIC
B4.1	B-F	PIPE TO VALVE WELD #92A	VOLUMETRIC	EXAMINATION. RELIEF REQUESTED TO DO SURFACE NDE INSPECTION.

RELIEF FOR CLASS I COMPONENTS  
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ITEM NO.	EXAMINATION CATEGORY TABLE IWB-2500	IDENTIFICATION OF WELD OR COMPONENT	INSPECTION REQUIRED BY IWB-2600	BASIS FOR REQUESTING RELIEF
B4.5	B-J	<u>10" MAIN STEAM PIPING</u> PIPE TO PIPE, WELD #19 PIPE TO PIPE, WELD #20 PIPE TO PIPE, WELD #21 PIPE TO PIPE, WELD #22  <u>MISCELLANEOUS PIPING COMPONENTS:</u> <u>MAIN STEAM AND SHUTDOWN CONDENSER</u> <u>PIPING SYSTEM HANGERS</u>	VOLUMETRIC	NO PERSONNEL ACCESS IN THIS AREA DUE TO THE SMALL INTERIOR OF THE PIPE CHASE CAVITY (2' SQUARE).  
B4.9	B-K-1	INTEGRALLY WELDED ATTACHMENT #MS-102	VOLUMETRIC	ACCESS RESTRICTIONS DUE TO PIPE CHASE CAVITY AND RADIATION LEVELS OF 1100 TO 1500 MR/HR GENERAL AREA.
B4.9	B-K-1	REMAINING INTEGRALLY WELDED ATTACHMENTS  <u>FEEDWATER AND CONDENSER CONDENSATE</u> <u>PIPING SYSTEM HANGERS</u>	VOLUMETRIC	BECAUSE OF WELD CONFIGURATION, RELIEF IS REQUESTED TO USE ALTERNATE INSPECTION METHOD UTILIZING SURFACE NDE.  
B4.9	B-K-1	INTEGRALLY WELDED ATTACHMENTS  <u>ALTERNATE CORE SPRAY PIPING SYSTEM</u> <u>HANGERS</u>	VOLUMETRIC	
B4.9	B-K-1	INTEGRALLY WELDED ATTACHMENTS	VOLUMETRIC	



RELIEF FOR CLASS I COMPONENTS  
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ITEM NO.	EXAMINATION CATEGORY TABLE IWB-2500	IDENTIFICATION OF WELD OR COMPONENT	INSPECTION REQUIRED BY IWB-2600	BASIS FOR REQUESTING RELIEF
B4.9	B-K-1	<u>DECAY HEAT SUCTION AND DISCHARGE PIPING SYSTEM HANGERS</u> INTEGRALLY WELDED ATTACHMENTS	VOLUMETRIC	BECAUSE OF WELD CONFIGURATION, RELIEF IS REQUESTED TO USE ALTERNATE INSPECTION METHOD UTILIZING SURFACE NDE. 
B4.9	B-K-1	<u>FORCED CIRCULATION SUCTION AND DISCHARGE HEADER PIPING HANGERS</u> INTEGRALLY WELDED ATTACHMENTS	VOLUMETRIC	
B4.9	B-K-1	<u>FORCED CIRCULATION SUCTION AND DISCHARGE AND PIPING SYSTEM HANGERS</u> INTEGRALLY WELDED ATTACHMENT	VOLUMETRIC	

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