



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
631 PARK AVENUE
KING OF PRUSSIA, PENNSYLVANIA 19406

July 29, 1980

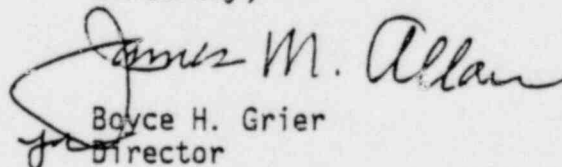
Docket No. 50-309

Maine Yankee Atomic Power Company
ATTN: Mr. Robert H. Groce
Senior Engineer - Licensing
25 Research Drive
Westborough, Massachusetts 01581

Gentlemen:

The enclosed Supplement to IE Information Notice No. 80-06 provides additional information with regard to an amendment to the Nuclear Regulatory Commission regulations requiring the immediate reporting by telephone of significant events at operating nuclear power plants. No written response is required. Should you have any questions regarding this information notice, please contact this office.

Sincerely,


Boyce H. Grier
Director

Enclosures:

1. Supplement to IE Information Notice No. 80-06 with Enclosure
2. List of Recently Issued IE Information Notices

CONTACT: D. L. Capton
(215-337-5266)

cc w/encls:
E. Wood, Plant Superintendent
E. W. Thurlow, President

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ENCLOSURE 1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

SSINS No.: 6870
Accessions No.:
8006190035

July 29, 1980

IE Information Notice No. 80-06
Supplement
Page 1 of 2

NOTIFICATION OF SIGNIFICANT EVENTS AT OPERATING POWER REACTOR FACILITIES

Description of Circumstances:

IE Information Notice No. 80-06 (IN 80-06) issued on February 27, 1980, forwarded a Federal Register Notice which indicated that the NRC regulations were amended by adding a new § 50.72, "Notification of significant events", to 10 CFR Part 50. The effective date of the new rule was February 29, 1980. This Information Notice supplements IN 80-06 and provides clarification on types of events that are required to be reported in accordance with the rule.

Public comments on the rule indicate the need for more detailed clarification on what occurrences are reportable in regard to certain event types listed in the rule. Commenters particularly stressed the need to establish more definitive thresholds for notifications of certain types of events. Experience so far with notifications being made in accordance with § 50.72 also suggest the need for clarification and more definitive guidance. The general categorization of some of the event types listed in the rule has resulted in notifications of events of less significance than was originally intended. As stated in the Federal Register Notice implementing § 50.72, the rule was intended to require immediate notification of "serious events that could result in an impact on the public health and safety."

The notification requirements in § 50.72 are currently being evaluated to determine actions needed in light of public comments on and experience with the rule. Notwithstanding future actions which may include some modifications to the rule and supplementary regulatory guidance, a short term need exists for clarifying the thresholds for notifications of certain categories of events. The following paragraphs provide clarification on what is reportable pursuant to the event categories listed in items (1), (7) and (9) of the rule.

Item (1) of § 50.72(a) requires notification of: "Any event requiring initiation of the licensee's emergency plan or any section of that plan." Notifications under this item refer to those initiating events or conditions that place the facility in a "Notification of Unusual Event" status. An unusual event indicates a potential degradation of the level of safety of the plant. Examples of initiating events or conditions that would be categorized as an unusual event are listed on the enclosed pages 4 and 5 of NUREG-0610, "Draft Emergency Action Level Guidelines for Nuclear Power Plants" (Appendix 1 to NUREG-0654, "Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants").

It is recognized that some overlap and duplication exists between the reporting requirements of item (1) and other items of § 50.72(a).

Item (7) of § 50.72(a) requires notification of: "Any event resulting in manual or automatic actuation of Engineered Safety Features, including Reactor Protection System." Actuation of Engineered Safety Features including the Reactor Protection System which result from and are part of the planned sequence during surveillance testing does not constitute an event reportable under this item.

Item (9) of § 50.72(a) requires notification of: "Any fatality or serious injury occurring on the site and requiring transport to an offsite medical facility for treatment." Serious injury is considered to be any injury that in the judgment of the licensee representative will require admission of the injured individual to a hospital for treatment or observation for an extended period of time (greater than 48 hours). Injuries that only require treatment and/or medical observation at a hospital or offsite medical facility, but do not meet the conditions specified above, are not required to be reported.

Enclosure:
Pages 4 and 5 of Appendix 1
to NUREG-0654

1. ECCS initiated
2. Radiological effluent technical specification limits exceeded
3. Fuel damage indication. Examples:
 - a. High offgas at BWR air ejector monitor (greater than 500,000 $\mu\text{ci}/\text{sec}$; corresponding to 16 isotopes decayed to 30 minutes; or an increase of 100,000 $\mu\text{ci}/\text{sec}$ within a 30 minute time period)
 - b. High coolant activity sample (e.g., exceeding coolant technical specifications for iodine spike)
 - c. Failed fuel monitor (PWR) indicates increase greater than 0.1% equivalent fuel failures within 30 minutes.
4. Abnormal coolant temperature and/or pressure or abnormal fuel temperatures
5. Exceeding either primary/secondary leak rate technical specification or primary system leak rate technical specification
6. Failure of a safety or relief valve to close
7. Loss of offsite power or loss of onsite AC power capability
8. Loss of containment integrity requiring shutdown by technical specifications
9. Loss of engineered safety feature or fire protection system function requiring shutdown by technical specifications (e.g., because of malfunction, personnel error or procedural inadequacy)
10. Fire lasting more than 10 minutes
11. Indications or alarms on process or effluent parameters not functional in control room to an extent requiring plant shutdown or other significant loss of assessment or communication capability (e.g., plant computer, all meteorological instrumentation)
12. Security threat or attempted entry or attempted sabotage
13. Natural phenomenon being experienced or projected beyond usual levels
 - a. Any earthquake
 - b. 50 year flood or low water, tsunami, hurricane surge, seiche
 - c. Any tornado near site
 - d. Any hurricane

14. Other hazards being experienced or projected
 - a. Aircraft crash on-site or unusual aircraft activity over facility
 - b. Train derailment on-site
 - c. Near or onsite explosion
 - d. Near or onsite toxic or flammable gas release
 - e. Turbine failure
15. Other plant conditions exist that warrant increased awareness on the part of State and/or local offsite authorities or require plant shutdown under technical specification requirements or involve other than normal controlled shutdown (e.g., cooldown rate exceeding technical specification limits, pipe cracking found during operation)
16. Transportation of contaminated injured individual from site to offsite hospital
17. Rapid depressurization of PWR secondary side.

RECENTLY ISSUED
IE INFORMATION NOTICES

Information Notice No.	Subject	Date Issued	Issued To
80-28	Prompt Reporting Of Required Information To NRC	6/13/80	All applicants for and holders of nuclear power reactor construction permits
80-27	Degradation of Reactor Coolant Pump Studs	6/11/80	All Pressurized Water Reactor Facilities holding power reactor OLs or CPs
80-26	Evaluation of Contractor QA Programs	6/10/80	All Part 50 Licensees
80-25	Transportation of Pyrophoric Uranium	5/30/80	Material Licensee in Priority/Categories II-A, II-D, III-I and IV-DI; Agreement State Licensees in equivalent categories
80-24	Low Level Radioactive Waste Burial Criteria	5/30/80	All NRC and Agreement State Licensees
80-23	Loss of Suction to Emergency Feedwater Pumps	5/29/80	All power reactor facilities with an OL or CP
80-22	Breakdown In Contamination Control Programs	5/28/80	All power reactor OLs and near term CPs
80-21	Anchorage and Support of Safety-Related Electrical Equipment	5/16/80	All power reactor facilities with an OL or CP
80-20	Loss of Decay Heat Removal Capability at Davis-Besse Unit 1 While in a Refueling Mode	5/8/80	All light water reactor facilities holding power reactor OLs or CPs