UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JUL 23 1980

Docket No. 50-341

APPLICANT: Detroit Edison Company

FACILITY: Enrico Fermi Atomic Power Plant, Unit 2

SUBJECT: SUMMARY OF JUNE 3, 1980 PLANT VISIT AND MEETING

On June 3, 1980, NRC staff and its consultants from Savanah River Laboratory, visited the Fermi 2 plant for the purpose of discussing descriptions and analyses of reactor systems contained in applicants Final Safety Analysis Report and viewing certain reactor systems.

A summary of the meeting and plant tour, prepared by T. Collins of the Reactor Systems Branch, is enclosed.

L. L. Kintner, Project Manager
Licensing Branch No. 2
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

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JUL 23 1980

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FERMI-2 MEETING SUMMARY

A meeting was held on June 3, 1980 at the Enrico Fermi Atomic Power Plant site to discuss questions generated by the Reactor Systems Branch (RSB), in the review of the Fermi-2 FSAR. The following organizations were represented: Detroit Edison Company (DECo), General Electric Company (GE), Savanah River Plant (SRP), and the Nuclear Regulatory Commission (NRC). Attachment 1 lists the attendees.

Two significant issues surfaced in the discussions. The first involves the amount of time which most be assumed prior to operator actions in transient and accident analyses. Section 6.3.2 of the FSAR states that no operator action is required for the first 10 minutes following ECCS actuation. Question 212.128 asks the impact of a 20 minute delay. DECo contended that the 10 minute mark was the accepted standard at the time when these analyses were submitted and that further analyses is therefore unjustified. The second issue involved the number of transients which must be reanalyzed using the ODYN Code. Q212.147 asks for all transients for which ODYN will provide more conservative results than the currently used REDY code. This would involve approximately 8 transients. DECo wishes to reanalyze only those 1 or 2 events which they believe to be most limiting based on REDY results. DECo has indicated that they wish to appeal these two issues prior to performing any additional analyses.

Several other questions were also addressed but involved only clarification for the applicant. A plant tour followed the discussions. The key areas of interest were a scale model of the NSSS, the ECCS components, Mark I containment and the plant thermal hydraulic systems. Attachment II details these systems.

ATTACHMENT I

ATTENDEES DECO - NRC

MEETING - JUNE 3, 1980

NRC

L. Kintner
T. Collins

EDISON

M. Batch
D. Lehnert
R. Anderson
L. Wooden
M. Deora
T. Okeefe
L. Schuerman
W. Colbert
E. Lusis

GE

J. Swan
G. Davis

SAVANAH RIVER PLANT

N. Hightower
N. Barnett
H. Reeve

ATTACHMENT II

FERMI-2 PLANT TOUR

1. Pump rooms: HPCI, LPCS, and LPCI (RHR) and should include discussions on: a) leakage detection; b) discharged line fill system; c) missile protection; and d) flooding of ECCS pump rooms.

2. Suppression pool, including discussions on: a) the torus water management system; b) downcomers (vent and safety/relief); c) the minimum pool level; and d) NPSH for the ECCS pumps.

3. Ultimate heat sink.

4. Safety/relief valves, including discussions on the method used to determine valve operation.

5. Control room, including discussions on the instrumentation made available for the operator to assess the need for ECCS operation.

6. Main Steam Supply System: a) outboard MSIVs; b) 52" steam manifold; c) main turbine stop valves (TSVs); d) main turbine control valves (TCVs); e) HP turbine; and f) LP turbines.

7. Turbine Bypass System: a) passive bypass piping and components (live steam to reheaters) - 10% capacity; and b) active bypass piping and components (EHC - Controlled Bypass System) - 26% capacity for both redundant loops.

8. Condensate and Feedwater System: a) main condenser containing first and second stage heaters; b) condenser pumps; c) condensate demineralizers; d) heater feed pumps; e) 3rd-5th stage heaters; f) reactor feed pumps; and g) 6th stage heaters.

9. RCIC System: a) steam driven turbine pump unit; b) barometric condenser and vacuum tank; c) condensate, vacuum and water leg pumps; d) full-flow test line to CST; and e) valves (ESI) F029 and (ESI) F031 in the pump suction line to the suppression)opening of these valves required for manual switchover to the suppression pool).

10. CRD System: a) hydraulic control units (scram and directional control valves and scram accumulator); b) scram discharge and instrument volumes; c) CRD return line modifications; d) stabilizing valves; and e) charging, cooling, drive, and exhaust headers.

11. Standby Liquid Control System: a) boron solution tank; b) test water tank; c) positive displacement pumps; and d) explosive valves.

12. Reactor Recirculating System (RRS): a) variable-speed motor-driven RRS pumps; b) motor-operated gate valves (suction and discharge); and c) motor-generator set to control RRS pump speed.