

FIGURE 2.1-2  
REACTOR CORE SAFETY LIMIT

TABLE 2.2-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION TRIP SETPOINTS

CRYSTAL RIVER - UNIT 3

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<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
1. Manual Reactor Trip	Not Applicable	Not Applicable
2. Nuclear Overpower	$\leq 105.5\%$ of RATED THERMAL POWER with four pumps operating	$\leq 105.5\%$ of RATED THERMAL POWER with four pumps operating
	$\leq 77.98\%$ of RATED THERMAL POWER with three pumps operating	$\leq 77.98\%$ of RATED THERMAL POWER with three pumps operating
3. RCS Outlet Temperature-High	$\leq 619^{\circ}\text{F}$	$\leq 619^{\circ}\text{F}$
4. Nuclear Overpower Based on RCS Flow and AXIAL POWER IMBALANCE <sup>(1)</sup>	Trip Setpoint not to exceed the limit line of Figure 2.2-1	Allowable Values not to exceed the limit line of Figure 2.2-1.
5. RCS Pressure-Low <sup>(1)</sup>	$\geq 1800$ psig	$\geq 1800$ psig
6. RCS Pressure-High	$\leq 2300$ psig	$\leq 2300$ psig
7. RCS Pressure-Variable Low <sup>(1)</sup>	$\geq (11.80 T_{\text{out}} - 5209.2)$ psig	$\geq (11.80 T_{\text{out}} - 5209.2)$ psig

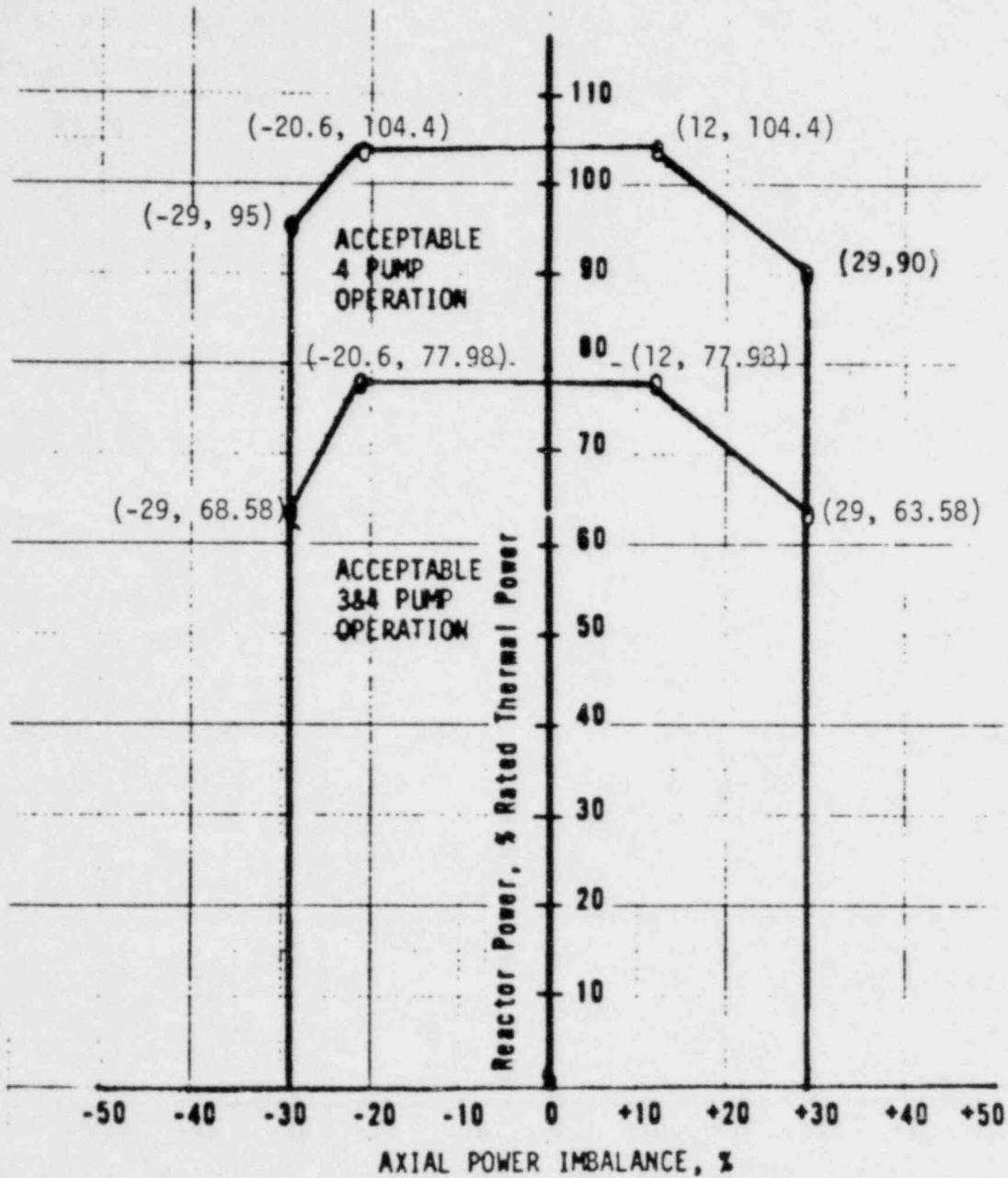


FIGURE 2.2-1

TRIP SETPOINT FOR NUCLEAR OVERPOWER BASED ON  
RCS FLOW AND AXIAL POWER IMBALANCE

## LIMITING SAFETY SYSTEM SETTINGS

### BASES

#### RCS Outlet Temperature - High

The RCS Outlet Temperature High trip  $<619^{\circ}\text{F}$  prevents the reactor outlet temperature from exceeding the design limits and acts as a backup trip for all power excursion transients.

#### Nuclear Overpower Based on RCS Flow and AXIAL POWER IMBALANCE

The power level trip setpoint produced by the reactor coolant system flow is based on a flux-to-flow ratio which has been established to accommodate flow decreasing transients from high power.

The power level trip setpoint produced by the power-to-flow ratio provides both high power level and low flow protection in the event the reactor power level increases or the reactor coolant flow rate decreases. The power level setpoint produced by the power-to-flow ratio provides overpower DNB protection for all modes of pump operation. For every flow rate there is a maximum permissible power level, and for every power level there is a minimum permissible low flow rate. Typical power level and low flow rate combinations for the pump situations of Table 2.2-1 are as follows:

1. Trip would occur when four reactor coolant pumps are operating if power is  $\geq 104.4\%$  and reactor flow rate is 100%, or flow rate is  $\leq 95.78\%$  and power level is 100%.
2. Trip would occur when three reactor coolant pumps are operating if power is  $\geq 77.98\%$  and reactor flow rate is 74.7%, or flow rate is  $\leq 71.84\%$  and power is 75%.

For safety calculations the maximum calibration and instrumentation errors for the power level were used.

## LIMITING SAFETY SYSTEM SETTINGS

### BASES

The AXIAL POWER IMBALANCE boundaries are established in order to prevent reactor thermal limits from being exceeded. These thermal limits are either power peaking kw/ft limits or DNBR limits. The AXIAL POWER IMBALANCE reduces the power level trip produced by the flux-to-flow ratio such that the boundaries of Figure 2.2-1 are produced. The flux-to-flow ratio reduces the power level trip and associated reactor power-reactor power-imbalance boundaries by 1.044% for a 1% flow reduction.

### RCS Pressure - Low, High and Variable Low

The High and Low trips are provided to limit the pressure range in which reactor operation is permitted.

During a slow reactivity insertion startup accident from low power or a slow reactivity insertion from high power, the RCS Pressure-High setpoint is reached before the Nuclear Overpower Trip Setpoint. The trip setpoint for RCS Pressure-High, 2300 psig, has been established to maintain the system pressure below the safety limit, 2750 psig, for any design transient. The RCS Pressure-High trip is backed up by the pressurized code safety valves for RCS over pressure protection, and is therefore set lower than the set pressure for these valves, 2500 psig. The RCS Pressure-High trip also backs up the Nuclear Overpower trip.

The RCS Pressure-Low, 1800 psig, and RCS Pressure-Variable Low,  $(11.80 T_{out}^{\circ}F - 5209.2)$  psig, Trip Setpoints have been established to maintain the DNB ratio greater than or equal to 1.30 for those design accidents that result in a pressure reduction. It also prevents reactor operation at pressures below the valid range of DNB correlation limits, protecting against DNB.

Due to the calibration and instrumentation errors, the safety analysis used a RCS Pressure-Variable Low Trip Setpoint of  $(11.80 T_{out}^{\circ}F - 5249.2)$  psig.

### 3/4.4 REACTOR COOLANT SYSTEM

#### REACTOR COOLANT LOOPS

#### LIMITING CONDITION FOR OPERATION

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3.4.1 Both reactor coolant loops and both reactor coolant pumps in each loop shall be in operation.

APPLICABILITY: As noted below, but excluding MODE 6.\*

#### ACTION:

##### MODES 1 and 2:

With one reactor coolant pump not in operation, STARTUP and POWER OPERATION may be initiated and may proceed provided THERMAL POWER is restricted to less than 77.98% of RATED THERMAL POWER and within 4 hours the setpoint for the Nuclear Overpower trip has been reduced to the value specified in Specification 2.2.1 for operation with three reactor coolant pumps operating.

##### MODES 3, 4 and 5:

- a. Operation may proceed provided at least one reactor coolant loop is in operation with an associated reactor coolant pump or decay heat removal pump.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.4.1 The Reactor Protective Instrumentation channels specified in the applicable ACTION statement above shall be verified to have had their trip setpoints changed to the values specified in Specification 2.2.1 for the applicable number of reactor coolant pumps operating either:

- a. Within 4 hours after switching to a different pump combination if the switch is made while operating, or
- b. Prior to reactor criticality if the switch is made while shut down.

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\* See Special Test Exception 3.10.3.

REACTIVITY CONTROL SYSTEMS

BORATED WATER SOURCES - SHUT-DOWN

LIMITING CONDITION FOR OPERATION

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3.1.2.8 As a minimum, one of the following borated water sources shall be OPERABLE:

- a. A concentrated boric acid storage system and associated heat tracing with:
  - 1. A minimum contained borated water volume of 6615 gallons.
  - 2. Between 11,600 and 14,000 ppm of boron, and
  - 3. A minimum solution temperature of 105°F.
- b. The borated water storage tank (BWST) with:
  - 1. A minimum contained borated water volume of 13,500 gallons.
  - 2. A minimum boron concentration of 2270 ppm, and
  - 3. A minimum solution temperature of 40°F.

APPLICABILITY: MODES 5 and 5.

ACTION:

With no borated water sources OPERABLE, suspend all operations involving CORE ALTERATION or positive reactivity changes until at least one borated water source is restored to OPERABLE status.

SURVEILLANCE REQUIREMENTS

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4.1.2.8 The above required borated water source shall be demonstrated OPERABLE:

- a. At least once per 7 days by:
  - 1. Verifying the boron concentration of the water.
  - 2. Verifying the contained borated water volume of the tank, and