



Portland General Electric Company

Donald J. Broehl Assistant Vice President

July 25, 1980

Trojan Nuclear Plant
Docket 50-344
License NPF-1

Mr. R. H. Engelken, Director
U. S. Nuclear Regulatory Commission
Region V
Suite 202, Walnut Creek Plaza
1990 N. California Blvd.
Walnut Creek, CA 94596

Dear Sir:

As stated in my letter of July 12, 1980, all actions required prior to resumption of power operations in accordance with my June 28, 1980 letter have been completed. Further, inasmuch as the Trojan Nuclear Plant resumed power operations on July 19, 1980, this letter constitutes the "report describing the results of our evaluations" to "be furnished to the NRC within 10 days after the resumption of power operation" committed to in my June 28 letter.

The following summarizes the results for heavyweight double wythe walls having safety significance exceeding the June 28, 1980 criteria and/or which were modified previously based upon LER 79-15, Supplement 4 criteria:

- 9 walls modified with throughbolts
- 2 walls modified with structural steel (1 cantilever)
- 1 wall removed from the STARDYNE model with equipment removed
- 12 heavyweight double wythe walls

The following summarizes the results for standard weight double wythe walls having safety significance exceeding June 28, 1980 criteria:

- 4 walls modified with structural steel (no cantilevers)
- 7 walls accepted for interim based on nonlinear displacement
— criteria or removed from STARDYNE
- 11 standard weight double wythe walls

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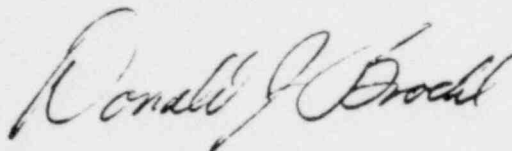
The following summarizes the results for single wythe walls having safety significance exceeding the June 28, 1980 criteria:

- 5 walls modified with structural steel (4 cantilevers)
- 10 walls accepted for interim based on nonlinear displacement
___ criteria and/or removed from STARDYNE model
- 15 single wythe walls

Two double wythe masonry walls with structural frequencies less than 20 cps provide support for safety-related equipment as defined in the June 28 letter. These walls (one normal weight and one heavy weight) support instrumentation (local indicators, transmitters and switches) which, although they form part of the system pressure boundary, do not have any control functions; thus, there is no adverse impact on the safety-related systems.

Those walls which have not yet been modified will be reevaluated with the interstory displacements from the finite element analysis. Those which exceed the 10 and 20 psi interwythe shear or bending criteria will be modified by October 31, 1980. Preliminary indications from the finite element analyses performed to date indicate that the predictions are well within the June 28, 1980 criteria.

Sincerely,



c: \ Mr. R. A. Clark, Chief
Operating Reactors Branch No. 3
Division of Licensing
U. S. Nuclear Regulatory Commission

Mr. Lynn Frank, Director
State of Oregon
Department of Energy