

Enclosure 2

VIEWGRAPHS PRESENTED BY PAS

TO THE

ACRS SUBCOMMITTEE ON TMI-2

OCTOBER 3, 1979

8007110490

OCTOBER 3, 1979

PRESENTATIONS TO ACRS SUBCOMMITTEE ON TMI #2

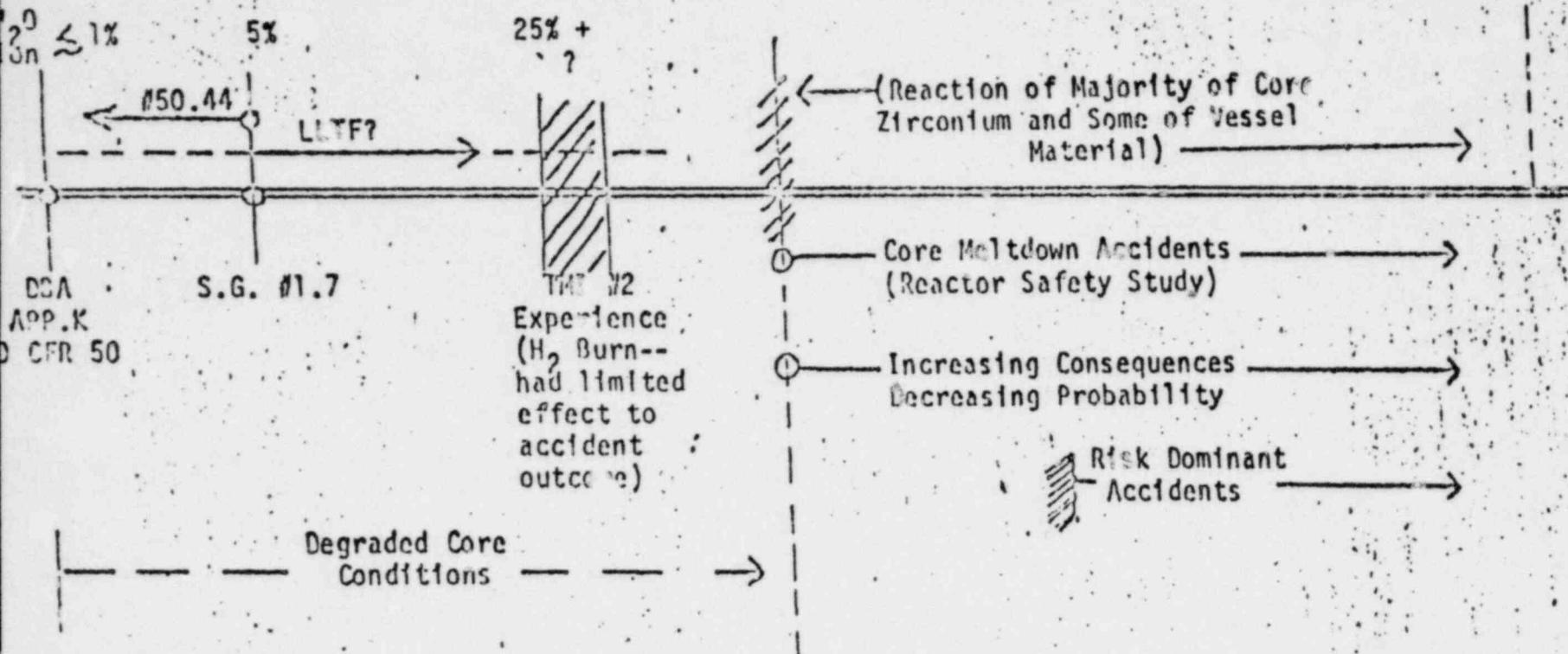
- ✓ o RISK-BASED INSIGHTS ABOUT POST-ACCIDENT  
HYDROGEN AND CONTAINMENT FAILURE MODES  
PAS/RES
- o RECENT WORK/ANALYSES ON STEAM EXPLOSIONS  
BCL  
AND HYDROGEN  
PAS/RES
- o STATUS OF RESEARCH PROGRAM ON CONTROLLED  
FILTERED VENTING OF CONTAINMENT  
PAS/RES

# 1

Present H<sub>2</sub>  
Control  
Philosophy  
(Licensing)

Concepts  
To Improve LWR Safety

e.g., Controlled Filtered  
Venting Concepts

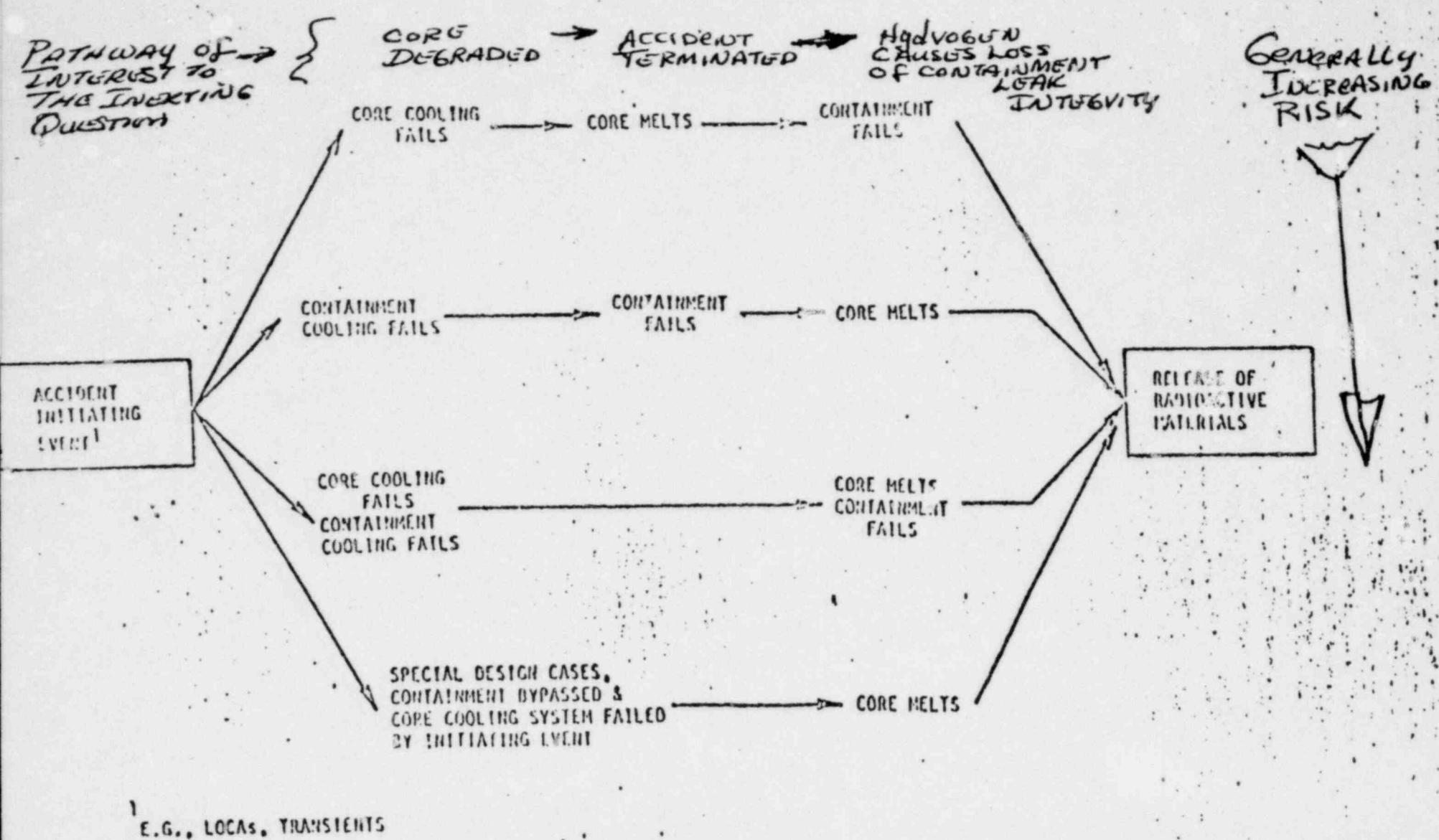


Reactor Safety Study Insights Can Be Useful to H<sub>2</sub> Control Questions.

- o Core Melt Accidents Most Likely to Occur from Transient and Small LOCA Initiated Events.
- o H<sub>2</sub> is But One Pathway to Containment Failure and Decisions on H<sub>2</sub> Control Should Be Considered in Suc. Context.

(#2)

## PRINCIPAL ACCIDENT SEQUENCES



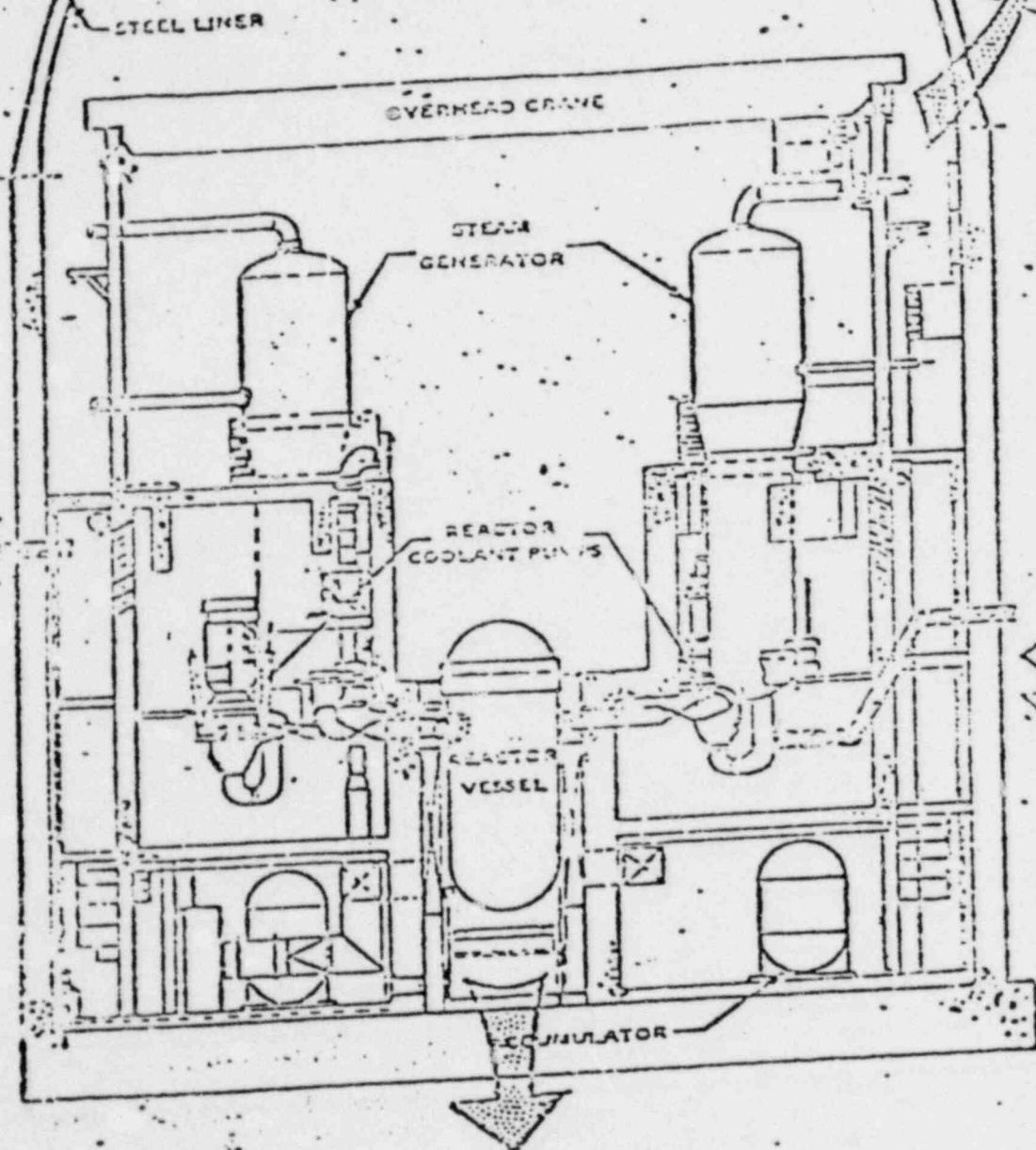
# 3

CAUSED FAILURE

FAILURE TO  
ISOLATE  
CONTAINMENT  
SYSTEM

04  
04 CONFINEMENT SPRAY → 10  
10

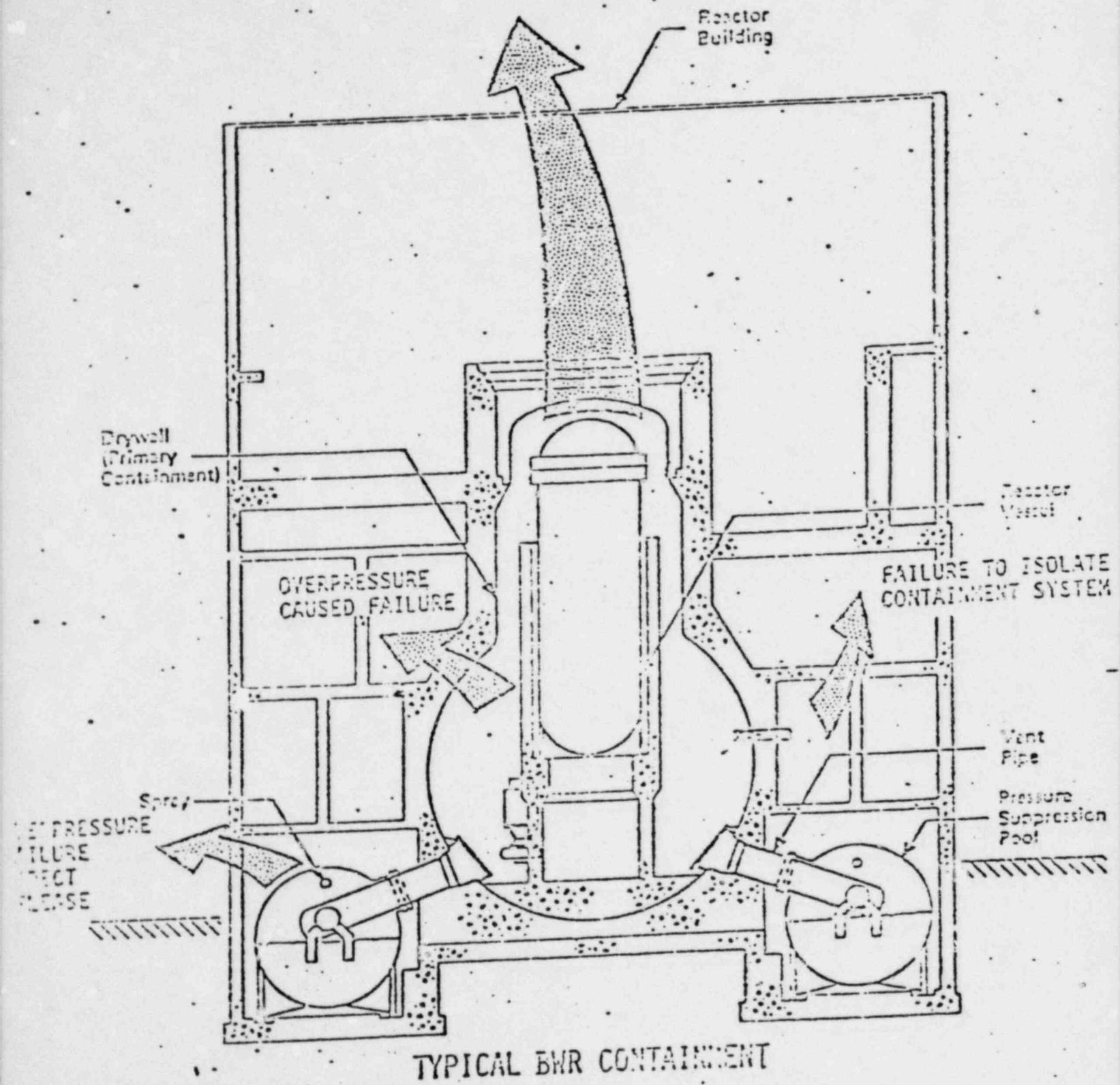
OVERPRESSURE  
FAILURE



PWR CONTAINMENT

44

VESSEL STEAM EXPLOSION  
CAUSED FAILURE



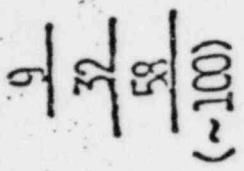
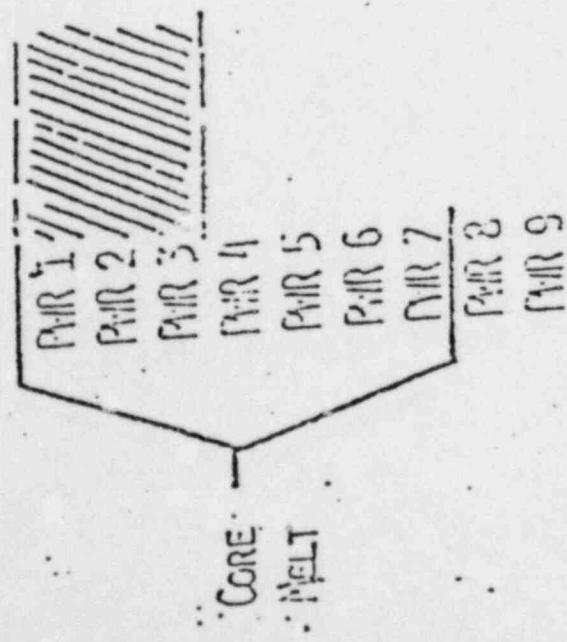
TYPICAL BWR CONTAINMENT

RELEASE CHARTS  
APPROXIMATE RISK CONSEQUENT ACCIDENTS

RELEASE CHARTS

APPROXIMATE RISK CONSEQUENT ACCIDENTS

FAMILY DEATHS      LATENT DEATHS



BASE: 100 DESEAS IN 1000000

#6

## PWR

CHECK VALVE RUPTURE  
 OVERPRESSURE  
 HYDROGEN BURNING  
 VESSEL STEAM EXPLOSION  
 MELT THROUGH  
 CONTAINMENT ISOLATION FAILURE  
 NO CONTAINMENT FAILURE

## EARLY FATALITIES

CHECK VALVE RUPTURE  
 OVERPRESSURE  
 HYDROGEN BURNING  
 VESSEL STEAM EXPLOSION  
 MELT THROUGH  
 CONTAINMENT ISOLATION FAILURE  
 NO CONTAINMENT FAILURE

## LATENT CANCER FATALITIES

## BWR

OVERPRESSURE DIRECT  
 OVERPRESSURE INDIRECT  
 VESSEL STEAM EXPLOSION  
 CONTAINMENT STEAM EXPLOSION  
 DRYWELL ISOLATION FAILURE  
 WETWELL ISOLATION FAILURE

## EARLY FATALITIES

OVERPRESSURE DIRECT  
 OVERPRESSURE INDIRECT  
 VESSEL STEAM EXPLOSION  
 CONTAINMENT STEAM EXPLOSION  
 DRYWELL ISOLATION FAILURE  
 WETWELL ISOLATION FAILURE

## LATENT CANCER FATALITIES

Relative Importance of Containment Failure Modes

(#7)

BASIS: LWR DESIGNS IN WASH-OFF

ANALYSIS: TWO DESIGNS IN MARCH 1960

#8

DEFINITIVE CONSEQUENCES TO 2907 CONTAINMENT FAILURE PRESSURE

5000
4000
3000
2000
1000
0

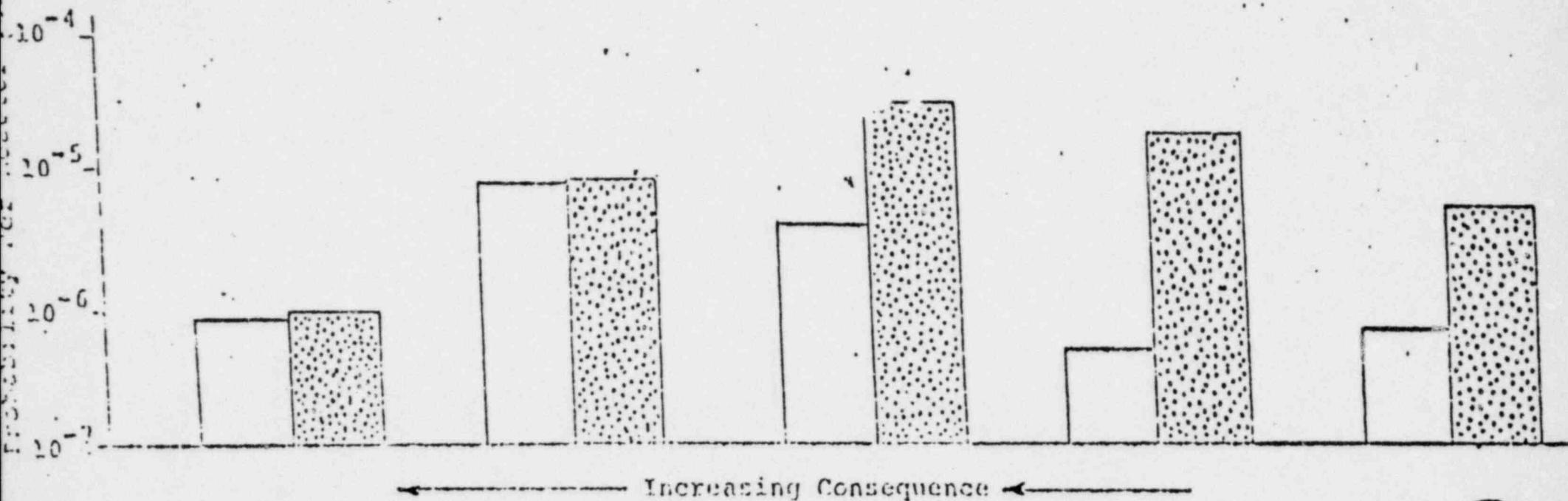
Note:  
Subsequent reveal's more  
radii's reveal's more  
that stream's thread  
that want indicated  
Contain 1400 indicated  
Contain 1400 indicated

DEFINITIVE CONSEQUENCES TO 2907 CONTAINMENT FAILURE PRESSURE

10000
8000
6000
4000
2000
0

Comparison of Dominant Accident Sequences: RSS Unit and Ice Condenser PWR

Category 1		Category 2		Category 3		Category 4		Category 5	
RSS	IC	RSS	IC	RSS	IC	RSS	IC	RSS	IC
None	S <sub>1</sub> H-Y	V	S <sub>2</sub> H-Y	S <sub>2</sub> C-S	S <sub>2</sub> H-Y	None	S <sub>1</sub> H-Y	None	S <sub>1</sub> D-Y
Dominant	$1 \times 10^{-7}$	$4 \times 10^{-6}$	$5 \times 10^{-6}$	$2 \times 10^{-6}$	$2 \times 10^{-5}$	Dominant	$1 \times 10^{-5}$	Dominant	$4 \times 10^{-6}$
	TMLB'-C	V			S <sub>1</sub> H-Y, S		S <sub>2</sub> D-Y		
	$2 \times 10^{-6}$	$9 \times 10^{-7}$			$3 \times 10^{-6}$		$.6 \times 10^{-6}$		
	TMLB'-Y				TML-Y				
	$7 \times 10^{-7}$				$3 \times 10^{-6}$				

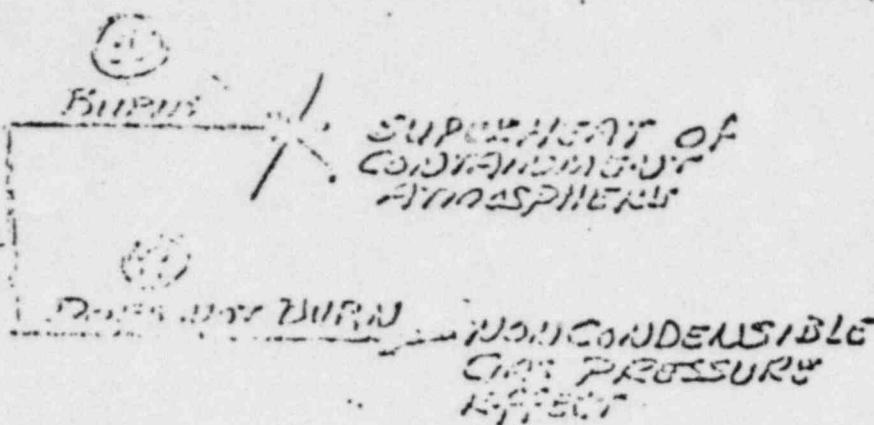


$\delta$  = Steam Over pressure failure  
 $\varphi$  = Steam + Hydrogen burn overpressure failure

#9

## CONCEPTUAL DESIGN OF PREVENTING H<sub>2</sub> BURN IN THE CONDENSER CONTAINMENT

九月九日



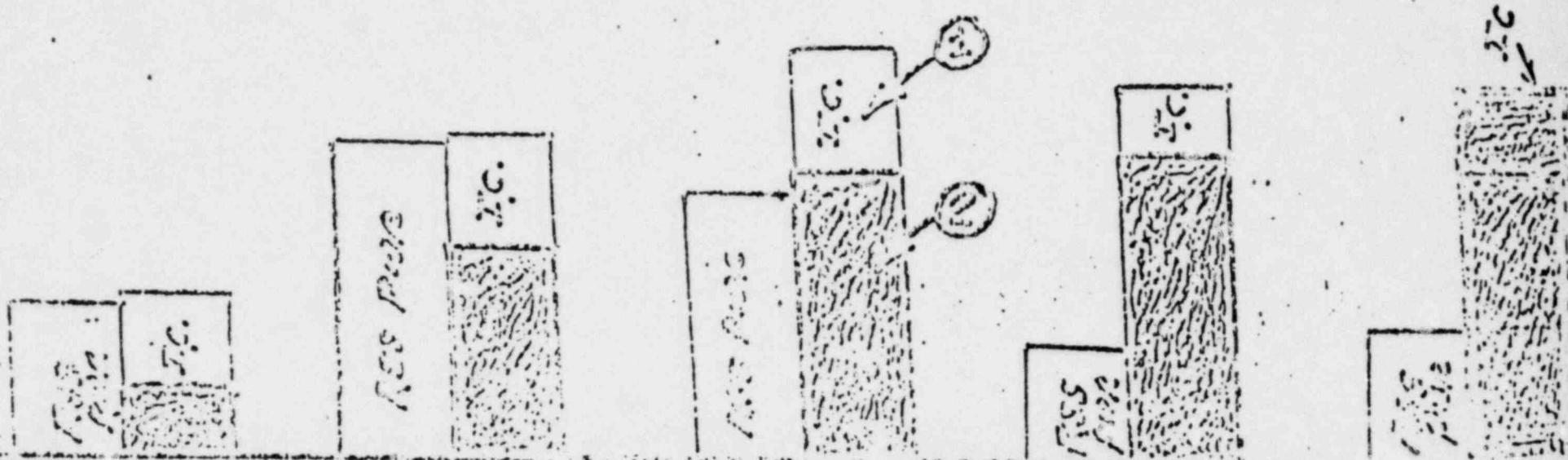
### **Category 1**

**Category 2**

**Category 3**

**Category 4**

**Category 5**



← Increasing Consequence →

#### R&D PWR and Test Condenser PWR

#10

PERSPECTIVES ON RSS-BWR DESIGN  
(BWR 4, MARK 1 CONTAINMENT, INERTED)

CONTAINMENT  
OVERPRESSURE  
FAILURE  
SCENARIOS

Transient followed by  
failure to shutdown,  
AWIS (TC)

Transient followed by  
failure of shutdown  
head removal system  
(TH)

Transient followed by  
failure to provide  
makeup water (TQUV)

Small LOCA followed  
by failure to provide  
makeup water (S<sub>2</sub>E)

ACCIDENT PROCESS PREDICTIONS\*

CONT. FAILS  
~77 MIN.

MELT STARTS  
~100 MIN.

MELT ENDS  
~144 MIN.

CONT. FAILS  
~2820 MIN.

MELT STARTS  
~3260 MIN.

MELT ENDS  
~3390 MIN.

START MELT  
~160 MIN.

END MELT  
~200 MIN.

CONT. FAILS  
~232 MIN.

START MELT  
~57 MIN.

END MELT  
~102 MIN.

CONT. FAILS  
~111 MIN.

POTENTIAL CONCEPTUAL VALUE

POTENTIAL RISK  
IMPACT  
OF SCENARIO

Large  
(Dominant)  
Sequence

INERTING

Negligible

CONTROLL  
VENT  
FILTE

Small  
to  
Moderat

Large  
(Dominant)  
Sequence

Negligible

Moderat  
to  
Large

Medium  
to  
Small

Small

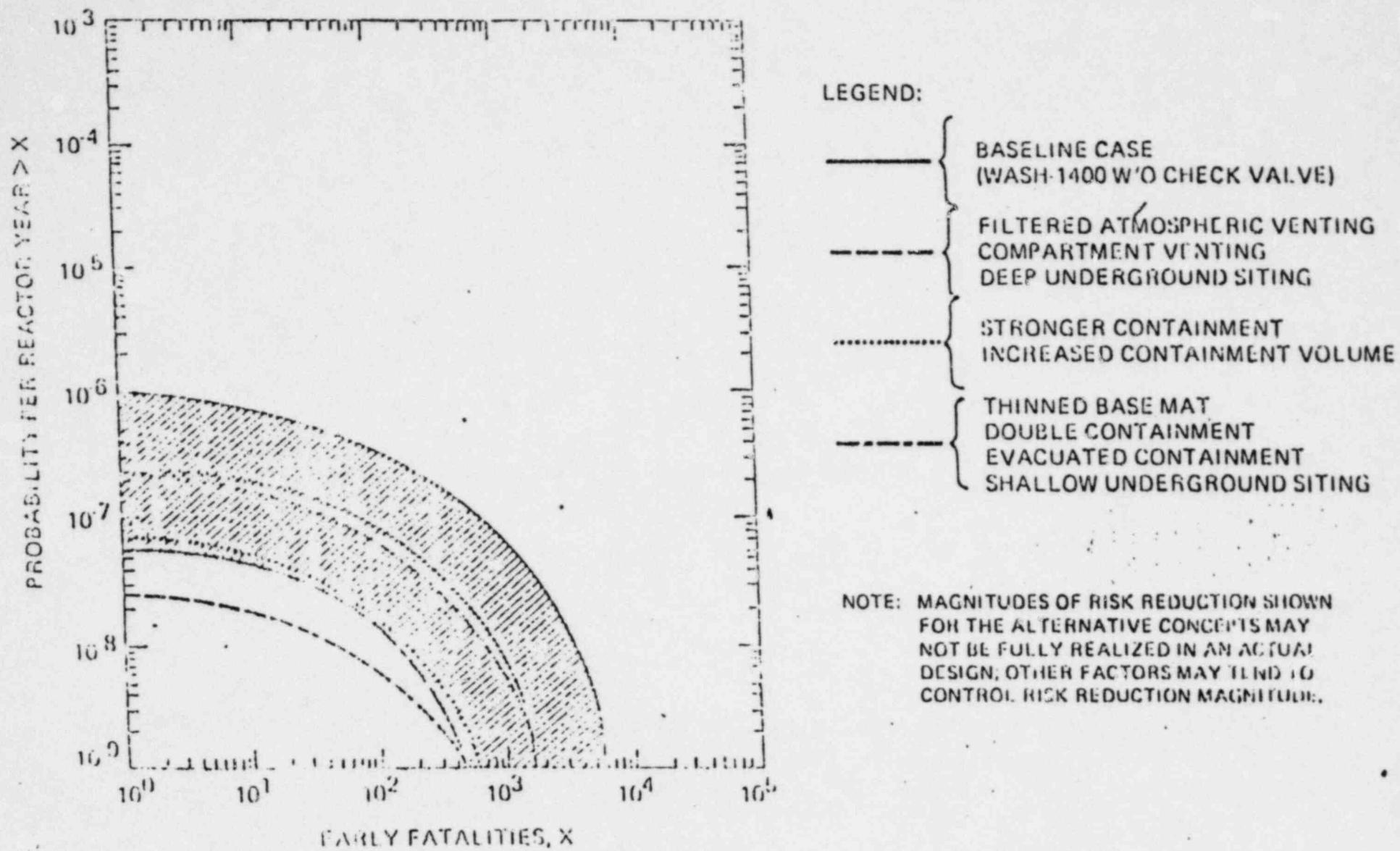
Moderat  
to  
Large

Medium  
to  
Small

Small

Small  
to  
Moderat

# ALTERNATIVE DESIGN CONCEPTS MAY SIGNIFICANTLY REDUCE RISKS FROM NUCLEAR ACCIDENTS



## SOME CONCLUDING OBSERVATIONS

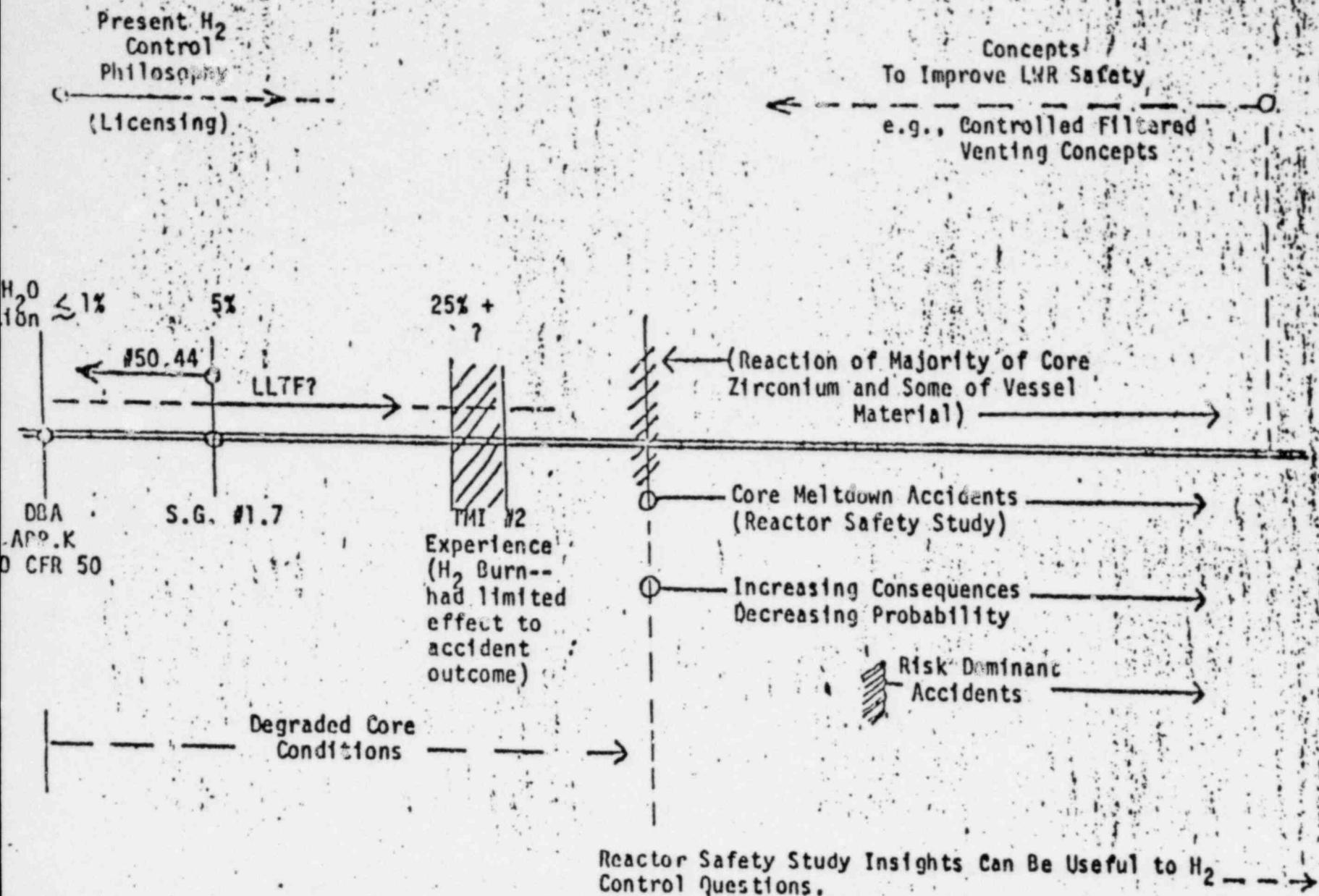
- o INERTING APPEARS TO HAVE SMALL VALUE IN REDUCING OVERALL ACCIDENT RISKS. IN SOME CASES, REDUCING ACCIDENT SEQUENCE PROBABILITY APPEARS TO HAVE EQUAL OR GREATER VALUE.
- o LARGER, HIGH DESIGN PRESSURE CONTAINMENTS ARE LESS SENSITIVE TO THE EFFECTS OF HYDROGEN.
- o H<sub>2</sub> CONTROL MEASURES THAT MAY BE ADOPTED PURSUANT TO THE H<sub>2</sub> SHOULD HAVE BENEFIT OF OVERALL RISK-BASED INSIGHTS AND CONTEXT.
- o WASH-140 EMPHASIZED CORE MELTDOWN ACCIDENTS; THE RISK REDUCTION BENEFITS OF CURRENT LICENSING H<sub>2</sub> CONTROL MEASURES FOR SUCH ACCIDENTS APPEAR SMALL.
- o RESEARCH ON IMPROVED LWR SAFETY CONCEPTS, E.G., CONTROLLED FILTERED VENTING SHOULD CONTINUE WITH PRIORITY SINCE SUCH CONCEPTS APPEAR TO HAVE RISK REDUCTION BENEFIT AND LOW-COST IMPACTS.

OCTOBER 3, 1979

PRESENTATIONS TO ACRS SUBCOMMITTEE ON TMI #2

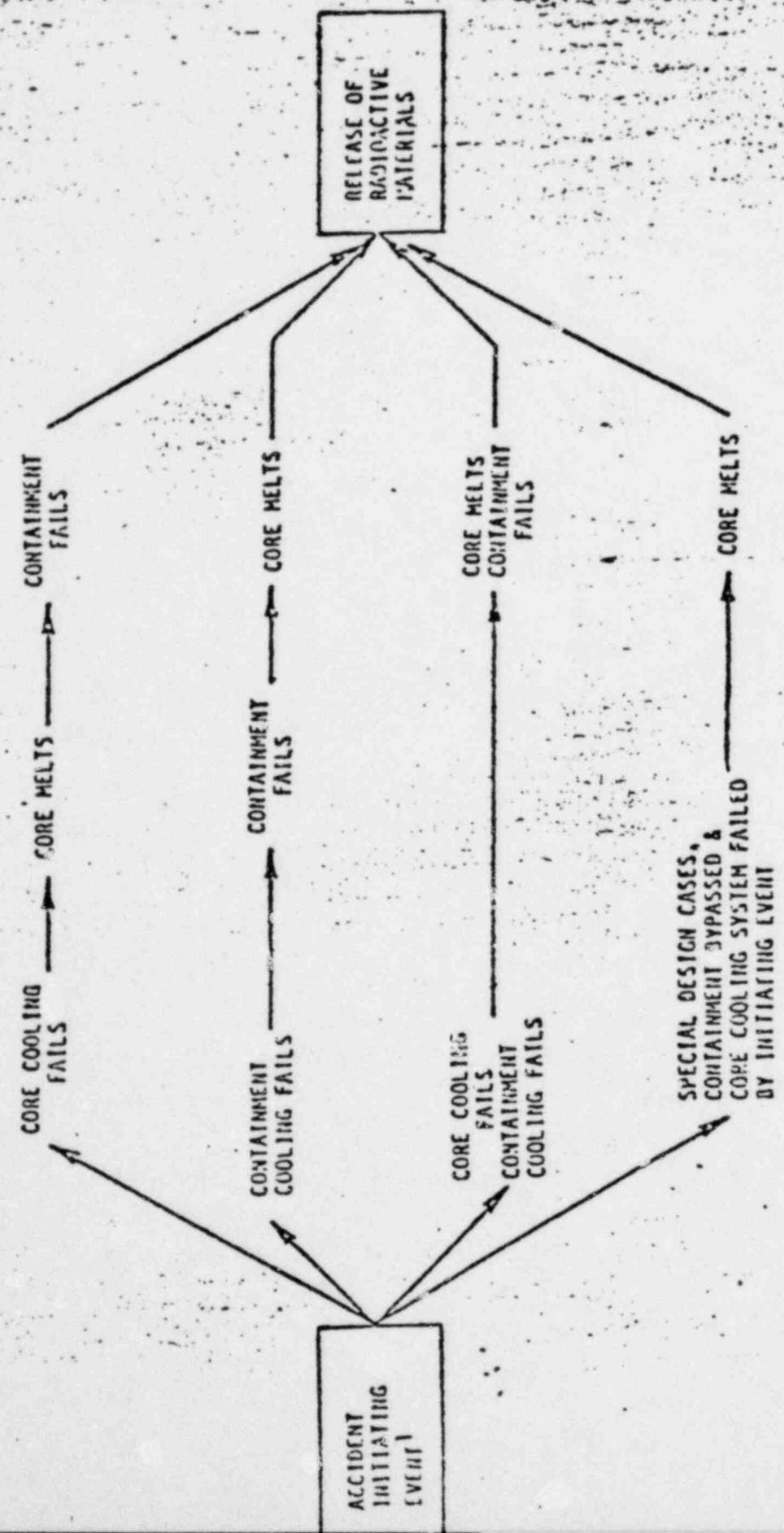
- ✓ O RISK-BASED INSIGHTS ABOUT POST-ACCIDENT HYDROGEN AND CONTAINMENT FAILURE MODES PAS/RES
- ✓ O RECENT WORK/ANALYSES ON STEAM EXPLOSIONS AND HYDROGEN BCL
- ✓ O STATUS OF RESEARCH PROGRAM ON CONTROLLED FILTERED VENTING OF CONTAINMENT PAS/RES

# BACKGROUND

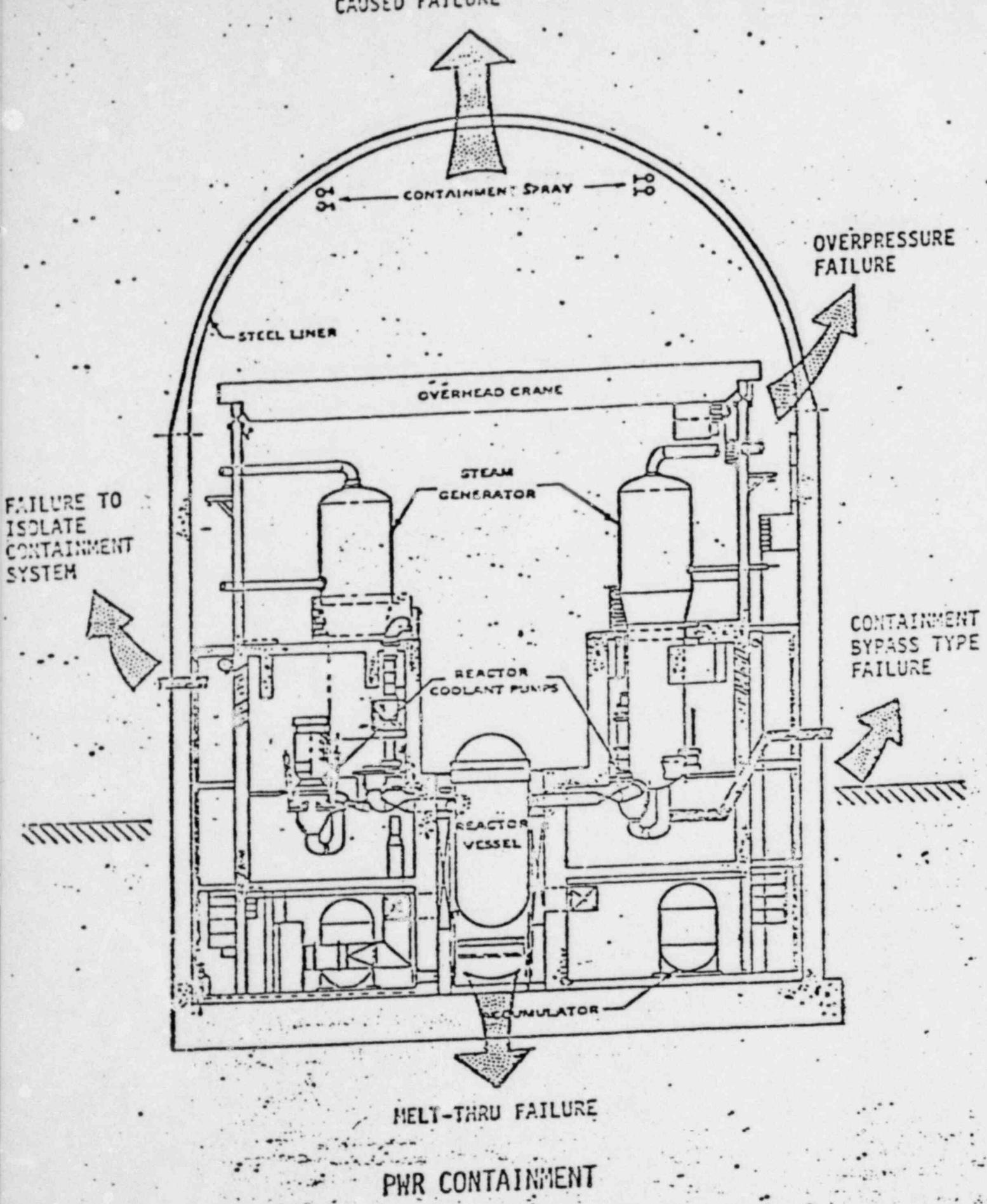


- o Core Melt Accidents Most Likely to Occur from Transient and Small LOCA initiated Events.
- o H<sub>2</sub> is But One Pathway to Containment Failure and Decisions on H<sub>2</sub> Control Should Be Considered in Such Context.

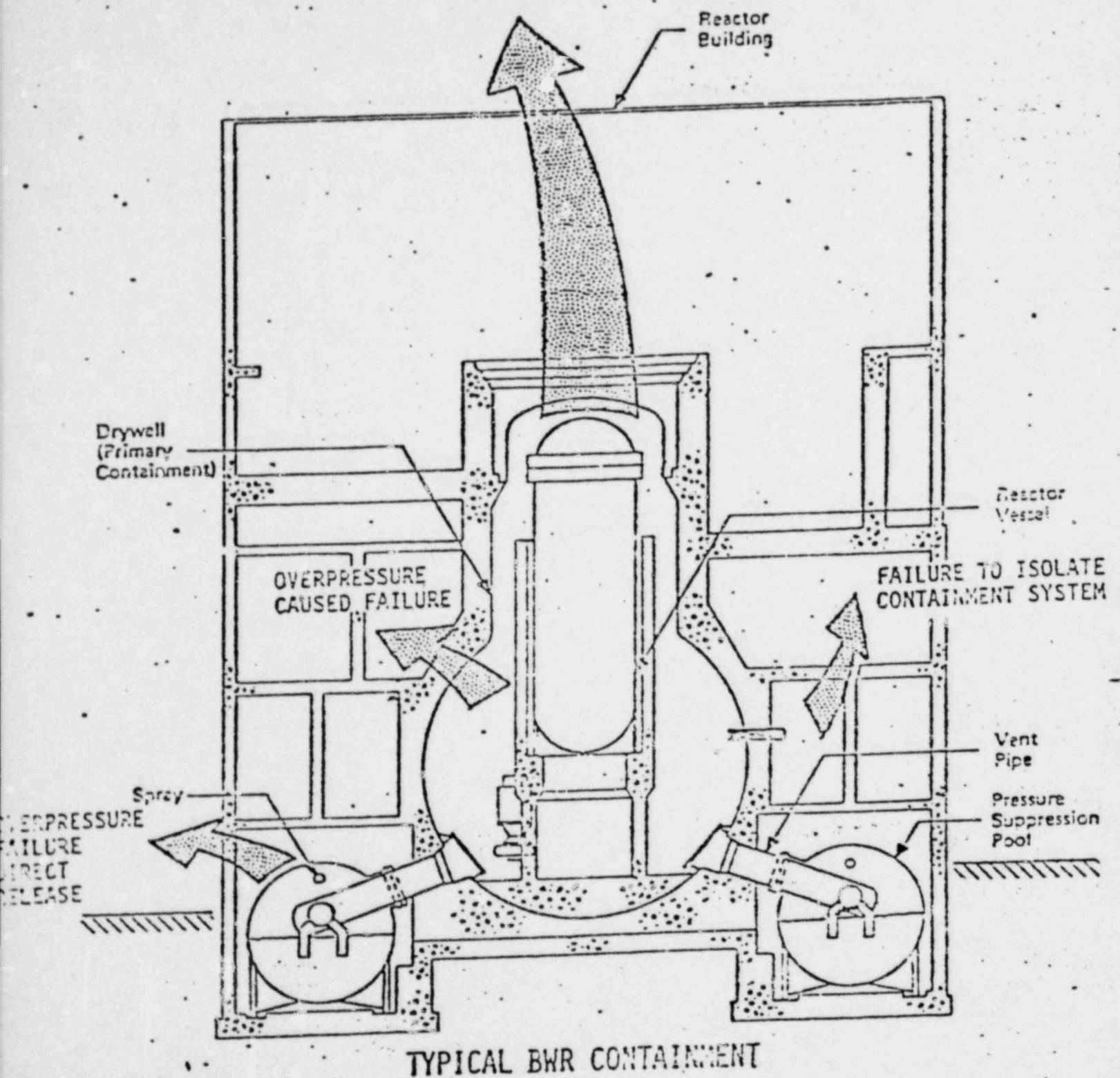
PRINCIPAL ACCIDENT SEQUENCES



VESSEL STEAM ELEVATION  
CAUSED FAILURE



VESSEL STEAM EXPLOSION  
CAUSED FAILURE



TYPICAL BWR CONTAINMENT

# RISK CONTRIBUTIONS FROM CORE-BELT ACCIDENTS

## RELEASE CATEGORIES

## APPROX. RISK CONTRIBUTION %

### EARLY DEATHS

### LATENT DEATHS

CORE  
MELT

PWR 1	/	/	/
PWR 2	/	/	/
PWR 3	/	/	/
PWR 4			
PWR 5			
PWR 6			
PWR 7			
PWR 8			
PWR 9			

35  
41  
24  
(~100)

9  
46  
39  
(~94)

CORE  
MELT

BWR 1	/	/	/
BWR 2	/	/	/
BWR 3	/	/	/
BWR 4			
BWR 5			

93  
7  
---  
(~100)

9  
32  
58  
(~100)

BASIS: LWR DESIGNS IN WASH-1'00

## PWR

CHECK VALVE RUPTURE  
 OVERPRESSURE  
 HYDROGEN BURNING  
 VESSEL STEAM EXPLOSION  
 MELT THROUGH  
 CONTAINMENT ISOLATION FAILURE  
 NO CONTAINMENT FAILURE

## EARLY FATALITIES

CHECK VALVE RUPTURE  
 OVERPRESSURE  
 HYDROGEN BURNING  
 VESSEL STEAM EXPLOSION  
 MELT THROUGH  
 CONTAINMENT ISOLATION FAILURE  
 NO CONTAINMENT FAILURE

## LATENT CANCER FATALITIES

OVERPRESSURE DIRECT  
 OVERPRESSURE INDIRECT  
 VESSEL STEAM EXPLOSION  
 CONTAINMENT STEAM EXPLOSION  
 DRYWELL ISOLATION FAILURE  
 WETWELL ISOLATION FAILURE

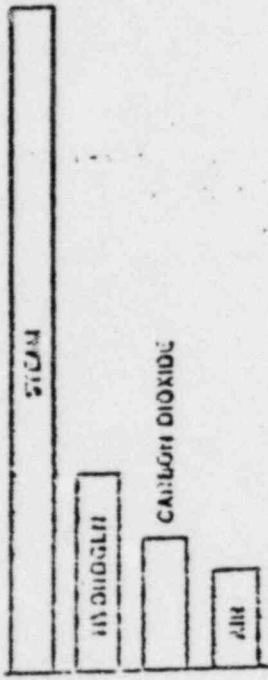
## EARLY FATALITIES

OVERPRESSURE DIRECT  
 OVERPRESSURE INDIRECT  
 VESSEL STEAM EXPLOSION  
 CONTAINMENT STEAM EXPLOSION  
 DRYWELL ISOLATION FAILURE  
 WETWELL ISOLATION FAILURE

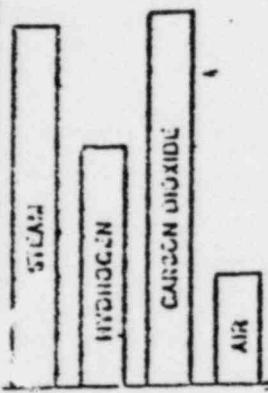
## LATENT CANCER FATALITIES

Relative Importance of Containment Failure Modes

RELATIVE CONTRIBUTIONS TO PVC CONTAINMENT FAILURE PRESSURE

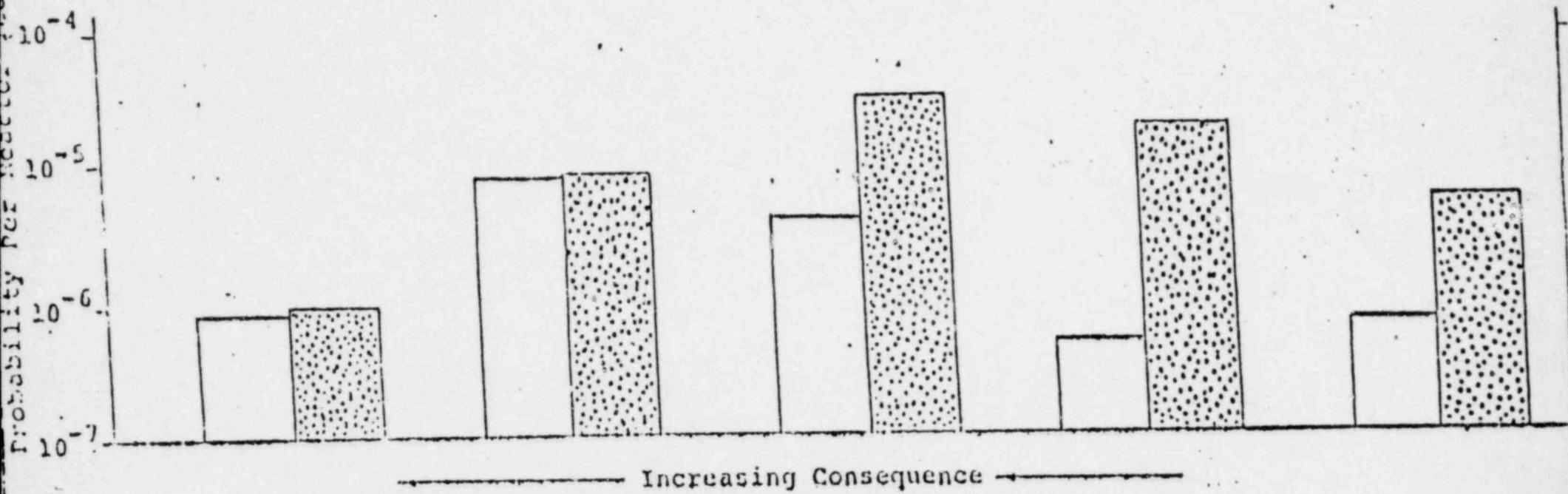


RELATIVE CONTRIBUTIONS TO LWR CONTAINMENT FAILURE PRESSURE

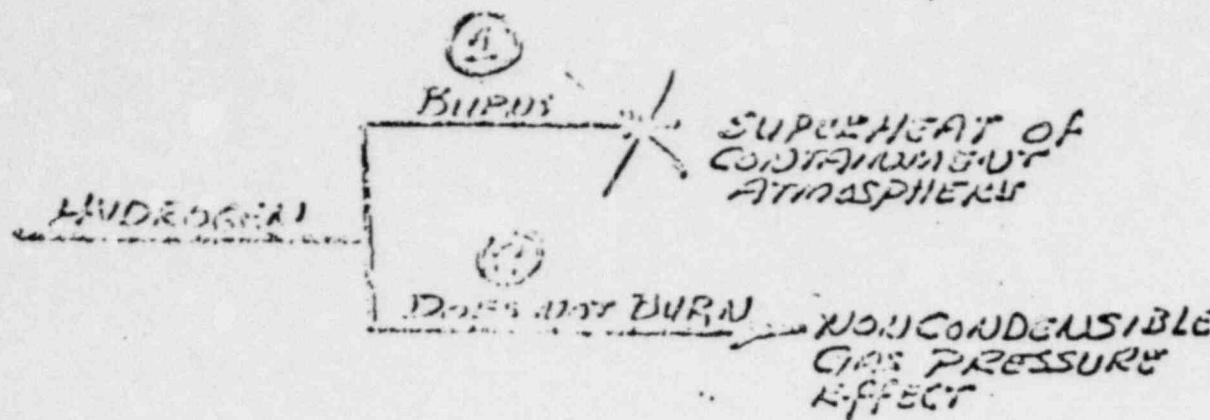


Comparison of Dominant Accident Sequences: RSS PWR and Ice Condenser PWR

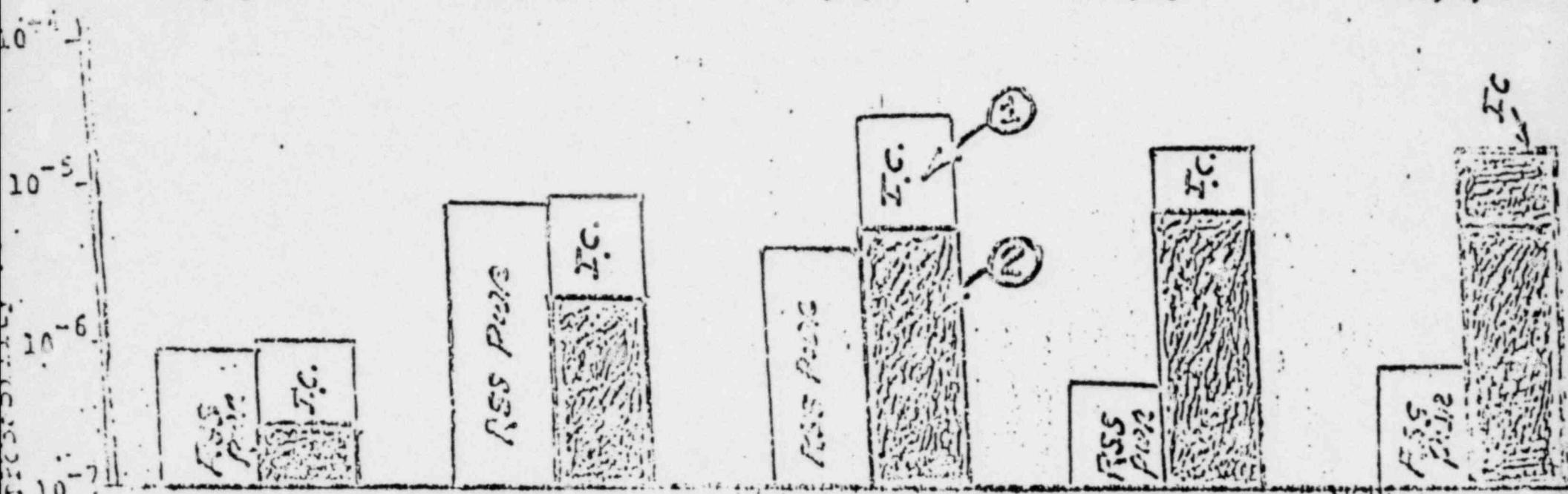
Category 1		Category 2		Category 3		Category 4		Category 5	
RSS	IC	RSS	IC	RSS	IC	RSS	IC	RSS	IC
None	S <sub>1</sub> H-a	V	S <sub>2</sub> H'F-Y	S <sub>2</sub> C-d	S <sub>2</sub> H-Y	None	S <sub>1</sub> H-Y	None	S <sub>1</sub> D-Y
Dominant	$1 \times 10^{-7}$	$4 \times 10^{-6}$	$5 \times 10^{-6}$	$2 \times 10^{-6}$	$2 \times 10^{-5}$	Dominant	$1 \times 10^{-5}$	Dominant	$4 \times 10^{-6}$
	THLB'-d	V			S <sub>1</sub> H'F-Y,d		S <sub>2</sub> D-Y		
	$2 \times 10^{-6}$	$9 \times 10^{-7}$			$3 \times 10^{-6}$		$.6 \times 10^{-6}$		
	TMLB'-Y				TML-Y				
	$7 \times 10^{-7}$				$3 \times 10^{-6}$				



CONCEPTUAL EFFECT OF PREVENTING H<sub>2</sub> BURN IN ICE CONDENSER CONTAINMENT



Category 1      Category 2      Category 3      Category 4      Category 5



Increasing Consequence

NSS PWR and Ice Condenser PWR

PERSPECTIVES ON RSS-BWR DESIGN  
 (UWR 4, MARK 1 CONTAINMENT, INERTED)

CONTAINMENT OVERPRESSURE FAILURE SCENARIOS	ACCIDENT PROCESS PREDICTIONS*	POTENTIAL CONCEPTUAL VALUE		
		POTENTIAL RISK IMPACT OF SCENARIO	INERTING	CONTROLL VENT FILTE
Transient followed by failure to shutdown, AIMS (TC)	CONT. FAILS ~77 MIN. → MELT STARTS ~100 MIN. → MELT ENDS ~144 MIN.	Large (Dominant) Sequence	Negligible	Small to Moderate
Transient followed by failure of shutdown heat removal system (TH)	CONT. FAILS ~2820 MIN. → MELT STARTS ~3260 MIN. → MELT ENDS ~3390 MIN.	Large (Dominant) Sequence	Negligible	Moderate to Large
Transient followed by failure to provide makeup water (TQUV)	START MELT ~160 MIN. → END MELT ~200 MIN. → CONT. FAILS ~232 MIN.	Medium to Small	Small	Moderate to Large
Small LOCA followed by failure to provide makeup water (S <sub>2</sub> E)	START MELT ~57 MIN. → END MELT ~102 MIN. → CONT. FAILS ~?? MIN.	Medium to Small	Small	Small to Moderate

PERSPECTIVES ON RSS-BWR DESIGN  
 (BWR 4, MARK 1 CONTAINMENT, INERTED)

CONTAINMENT  
OVERPRESSURE  
FAILURE  
SCENARIOS

Transient followed by  
failure to shutdown,  
ATWS (TC)

Transient followed by  
failure of shutdown  
heat removal system  
(THS)

Transient followed by  
failure to provide  
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Small LOCA followed  
by failure to provide  
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~77 MIN.

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START MELT  
~160 MIN.

END MELT  
~200 MIN.

CONT. FAILS  
~232 MIN.

START MELT  
~57 MIN.

END MELT  
~102 MIN.

CONT. FAILS  
~113 MIN.

POTENTIAL CONCEPTUAL VALUE

POTENTIAL RISK  
IMPACT  
OF SCENARIO

Large  
(Dominant)  
Sequence

Large  
(Dominant)  
Sequence

Medium  
to  
Small

Medium  
to  
Small

INERTING

Negligible

Negligible

Small

Small

CONTROLL  
VENT  
FILTE

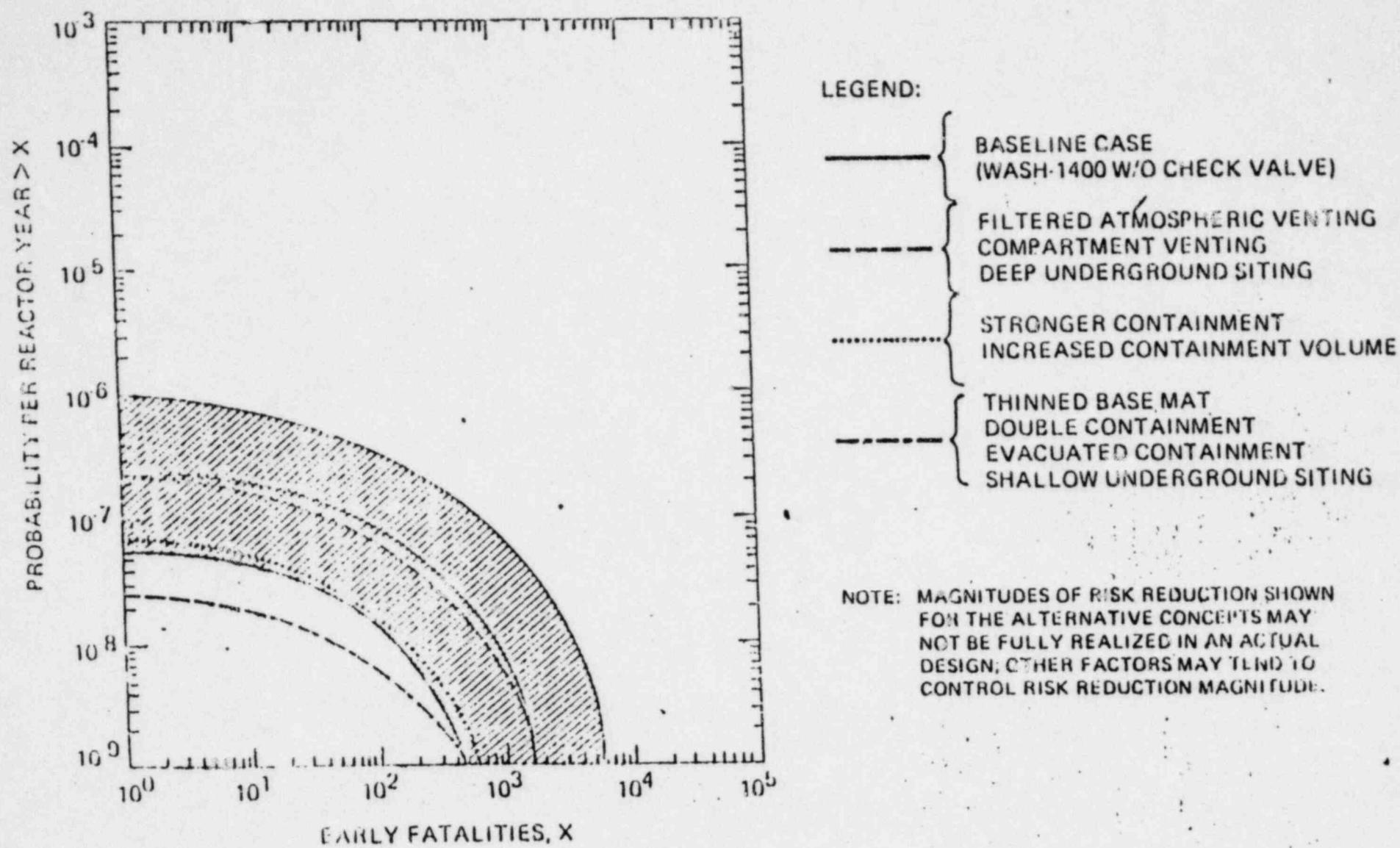
Small  
to  
Modera

Modera  
to  
Large

Modera  
to  
Large

Small  
to  
Modera

## ALTERNATIVE CONTAINMENT CONCEPTS MAY SIGNIFICANTLY REDUCE RISKS FROM NUCLEAR ACCIDENTS



## SOME CONCLUDING OBSERVATIONS

- o INERTING APPEARS TO HAVE SMALL VALUE IN REDUCING OVERALL ACCIDENT RISKS. IN SOME CASES, REDUCING ACCIDENT SEQUENCE PROBABILITY APPEARS TO HAVE EQUAL OR GREATER VALUE.
- o LARGER, HIGH DESIGN PRESSURE CONTAINMENTS ARE LESS SENSITIVE TO THE EFFECTS OF HYDROGEN.
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