Docket Nos. 50-329 and 50-330

July 14, 1971

Consumers Power Company ATTN: Mr. Russell Youngdahl Senior Vice President 212 West Michigan Street Jackson, Michigan 49201

THIS DOCUMENT CONTAINS POOR QUALITY PAGES

Gentlemen:

On June 19, 1971 the AEC adopted interim acceptance criteria for the performance of emergency core cooling systems (ECCS) in light-water nuclear power plants. A copy of the Commission's interim policy statement on this matter is enclosed for your information. In accordance with Section IV.B of the interim policy statement, before we can complete our evaluation of the ECCS for the Midland Plant Units No. 1 and 2 we need information to show that the system meets the general criteria of Section IV.A. using a suitable evaluation model. We are continuing discussions with representatives of the Babcock & Wilcox Company (B&W) that are directed toward establishing a suitable evaluation model using B&W computer codes for evaluation of plants incorporating a nuclear steam supply system designed by B&W.

The information that we need regarding analyses performed with a suitable evaluation model is:

- (1) For the break size range, location and type mentioned in Appendix A, Part 1, of the interim policy statement, provide information pertaining to (a) the system pressure, (b) the hot-spot clad temperature, local mass velocity, fluid temperature, and heat transfer coefficient, (c) the core pressure drop, quality, and mass velocity, (d) the heat flux distribution in the hot channel, (e) the flow rates in the upper and lower plenums, (f) the flow rates in the broken and intact cold-leg and hot-leg piping, (g) the flow rate out of the break, and (h) percent clad metal-water reaction.
- (2) Provide a detailed discussion of the calculation used to predict heat transfer during the reflood portion of the transient.

OFFICE >						
SURNAME >	80	06160	390	>	1	
DATE		***************************************			¥	· · · · · · · · · · · · · · · · · · ·

(3) Discuss in detail any deviations in the evaluation model used in the foregoing studies from that described in Appendix A, Part 1 of the interim policy statement, if that model is used.

In addition, you should submit for our review any changes to the Technical Specifications for the plant that may be required on the basis of the results of your analyses.

When you have obtained the required information, please send us 60 copies. When we have completed our review of this information, we will submit our assessment to the Midland Plant Atomic Safety and Licensing Board and provide copies to all parties in this proceeding.

Sincerely,

Original signed by Peter A. Morris

Peter A. Morris, Director Division of Reactor Licensing

Enclosure: AEC Interim Policy Statement

Distribution: Docket File AEC PDR DR Reading DRL Reading PWR-1 Reading CKBeck, DR MMMann, DR FSchroeder, DRL RSBoyd, DRL RCDeYoung, DRL DSkovholt, DRL TRWilson, DRL EGCase, DRS RRMaccary, DRS RWKlecker, DRL B/C's, DRL/DRS Lic. Asst. OGC ACRS (16) W. Nyer (2) Seismic Design Consultant

X-724 ber All	
SURNAME MULTer sp De Young Schrøeder Morris	
DATE > 7/9/71 7/34 /71 7/ 171 7/17/71	