

Docket No. 50-302

Florida Power Corporation
ATTN: Mr. J. T. Rodgers, Assistant Vice President
and Nuclear Project Manager
P. O. Box 14042
St. Petersburg, Florida 33733

Gentlemen:

OPERATIONAL QA PROGRAM - CRYSTAL RIVER UNIT NO. 3

We have evaluated the enclosure transmitted with your September 22, 1976, letter. The enclosure which documents your "comparison analysis" of the Operational QA Program for Crystal River Unit No. 3 (CR-3) with the requirements of ANSI N18.7 - 1976 does not provide the necessary information requested in W. Butler's June 25, 1976, letter to you.

We would like to emphasize again that all nuclear power stations have been requested to describe in the docket how their operational QA program satisfies either ANSI N18.7 - 1976 and Regulatory Guides Nos. 1.8, 1.30, 1.37, 1.38, 1.39, 1.54, 1.58, 1.64 - Rev. 1, 1.74, 1.88, and 1.94 or the WASH documents Nos. 1283, 1309, and 1284. The controls in these documents, which in the case of N18.7 is very recent, define acceptable ways to assure conformance with Appendix B criteria; provide a clear basis for NRC acceptance; and assure that the docketed QA program is consistent with good practice as defined in the appropriate industry standards.

Your September 22 response relative to our request for Florida Power Corporation (Crystal River Unit No. 3 - Docket No. 50-302) to commit to comply with ANSI 18.7 - 1976 and the Regulatory Guides (Nos. 1.8, 1.30, 1.37, 1.38, 1.39, 1.54, 1.58, 1.64 - Rev. 1, 1.74, 1.88, and 1.94) which endorse the ANSI standards referenced in N18.7 - 1976 does not provide us with a sufficient understanding of the degree to which you will implement the controls in these documents. Accordingly, we request you provide us with a clear statement regarding the degree to which you will implement the controls in these documents by means of: A specific commitment to comply with the above documents or to the WASH documents Nos. 1283, 1309, and 1284, as previously addressed in our letter of June 25, 1976; and identification of any alternative controls, or exceptions, with appropriate supporting

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information.

In addition, the descriptive changes contained in "Notes" Nos. 1, 5, 6, 7, 9, 10, and 12 of the enclosure were found to be acceptable, and the appropriate sections of the FSAR should be amended accordingly. The above information should be submitted within 30 days following receipt of this letter.

We are available for a meeting to discuss this subject if you so desire.

Sincerely,

John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management

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