Docket No. 50-346	DISTRIBUTION Docket File NRC PDR Local PDR LWR 2-3 Rdg VAMoore RDeYoung	ACRS (14) JPanzarella JRBuchanan, ORNL TBAbernathy, DTIE
Toledo Edison Company ATTN: Mr. Lowell E. Roe Vice President Facilities Development 300 Edison Plaza Toledo, Ohio 43652	FSchroeder AKenneke MWilliams RKlecker OELD OI&E (3) LEngle EGoulbourne	POOR ORIGINAL
Gentlemen:	ASchwencer	

We request that the two items provided in the enclosure to this latter be addressed as part of the Appendix K ECCS submittal for the Davis Besse Nuclear Power to tion, Unit 1.

Mr. L. Engle, the Licensing Program Manager for Daviz Besse, 1 it 1, has already contacted Mr. C. Novak and informed him of this need for additional information concerning your ECCS submittal.

In order to maintain our licensing schedule, we will need your response to the enclosure by July 9, 1975. If you cannot meet this date, please inform us within seven days after receipt of this letter so that we may revise our scheduling.

Please contact us if you have any questions regarding the enclosure provided.

Sincerely,

Original Signed by A. Schwencer

A. Schwencer, Chief Light Water Reactors Branch 2-3 Division of Reactor Licensing

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Enclosure: Request for additional information concerning ECCS submittal

cc: See next page

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ORIGINAL

Toledo Edison Company

- 2 -

cc: Donald H. Hauser, Esquire
The Cleveland Electric Illuminating Co.
P. O. Box 5000, Room 610
Cleveland, Ohio 44101

periode.

Gerald Charnoff, Esquire
Shaw, Pittman, Potts, Trowbridge
and Madden
910 - 17th Street, NW
Washington, D. C. 20006

Leslie Henry, Esquire Fuller, Seney, Henry & Hodge 300 Madison Avenue Toledo, Ohio 43604

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Form ABC-318 (Rev. 9-53) ABCM 0240				

ENCLOSURE

REQUESTS FOR ADDITIONAL INFORMATION APPENDIX K ECCS SUBMITTAL FOR THE DAVIS BESSE NUCLEAR POWER STATION, UNIT 1 DOCKET NO. 50-346

- If Davis-Besse will operate with a part-pump reactor coolant system configuration, this capability must be supported by identifying and analyzing the worst break size and location (i.e., idle loop versus operating loop). In addition, sufficient justification must be provided to conclude that the shape of the PCT versus Break Size curve would not be significantly after ed by the partial loop configuration. (A partial loop configuration - defined where one or more reactor coolant pumps are not operating.)
- 2. It has recently come to our attention that after a LOCA, the operability of certain critical systems may be jeopardized due to the location of certain related components (such as valve motors, busses, etc.) at or near the containment floor. Post-LOCA flooding of the containment could submerge these components and could involve such essential safety features as the safety injection system, containment isolation network, or systems needed to limit boric acid concentration in the reactor vessel during long-term core cooling. The following information must be supplied:
 - a) Whether or not any components, such as valve motors or electric busses, will be submerged following a LOCA in Davis-Besse 1.
 - b) If it is determined that such components will be flooded;
 - 1. Identify this equipment.
 - Evaluate the potential consequences of flooding of this equipment for both the short term and long term. (Long term should include consideration of the potential problem of excessive concentrations of boric acid).
 - 3. Propose design changes to solve the potential flooding problem.