Docket Nio. 50-346

Toledo Bdison Company
ATTN: Mr. Lowell E. Roe Vice President
Facilities Development
300 2dison Plaxa Toledo, ohio 43652

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Gentieren:
To request that the two items provicied in the enclosure to this lutter be addressed as part of the Appendix K ECCS subnittal for the Davis 3esse Nuclear Power : tion, Unit 1.
fr. L. Sngle, the Licensing Program Manager for Davi= Besse, it i, has dircady contacted $: 1 r$. C. Sovaic and informed hin of ilis need for additionai information concerning your ECCS subaittal.

In order to maintain our 1 fecnatng scinedule, we will need your response to the enclosure by Juiy 9, 1975. If you cannot geet this date, please inform us within seven days after receipt of this ietter so that we nay revise our icheduilng.

Please contact is $1:$ you have any questions regarding the anclosure providec.

Sincerely,
Original signed by
A. Schwencer
A. Schwencer, Chies

Light Water Veactors Sranch 2-3
Division of Reactor Licensing

> Enclosure:
> Request for additional
> Information concerning ECCS aubnittal

cc: Sce next paze

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Toledo Edison Cotapany
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ce: Donald H. Hauser, Esquire
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## ENCLOSURE

REQUESTS FOR ADDITIONAL INFORMATION APPENDIX K ECCS SUBMITTAL FOR THE
davis besse nuctear poner station, UNIT 1
DOCKET NO. 50-346

1. If Davis-Besse will operate with a part-pump reactor coolant system config cation, this capability must be supported by identifying and analyzing the worst break size and location (i.e., idle loop versus operating loop). In addition, su-icient justification must be provided to conclude that the shape of the PCT versus Break Size curve would not be significantly $a^{\text {' te ed }}$ by the partial loop configuration. (A partial loop configuratio. defined where one or more reactor coolant pumps are not operating.)
2. It has recently come to our attention that after a LOCA, the operability of certain critical systems may be jeopardized due to the location of certain related components (such as valve notors, busses, etc.) at or near the containment floor. Post-LOCA flooding of the containment could submerge these components and could involve such essential safety features as the safety injection system, containment isolation network, or systems needed to limit boric acid concentration in the reactor vessel during long-term core cooling. The following information must be supplied:
a) Whether or not any components, such as valve motors or electric busses, will be submerged following a LOCA in Davis-Resse 1.
b) If it is determined that such components will be flooded;
3. Identify this equipment.
4. Evaluate the potential consequences of flooding of this equipment for both the short term and long term. (Long term should include consideration of the.potential problem of excessive concentrations of boric acid).
5. Propose design changes to solve the potential flooding problem.
