17-771 LICENSEE EVENT REPORT
CONTROL BLOCK:
0 H D B S 1 2 Ø Ø - Ø Ø P F - Ø 3 3 4 1 1 1 1 1 2 57 CAT 58
CON'T REPORT L 6 0 5 0 - 0 3 4 6 7 0 13 2 5 7 8 8 0 4 2 0 7 8 9
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) [0 2
SG 1-2 level continued to increase even after closing the Startup Feedwater Valve.
O 4 At 0455 hours on 3/25/78, the level exceeded the maximum limit. This placed the unit
0 5 in Action B of T.S. 3.4.5. There was no danger to the health and safety of the public
o or unit personnel. No steam line break occurred during the short period of time the
Steam Generator level slightly exceeded 86%. (NP-33-78-36)
0 8 1
SYSTEM CAUSE CODE CODE SUBCODE COMPONENT CODE SUBCODE
Component Comp
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27 [1 0 An operator entered containment and opened one SG drain on SG 1-2. This caused the
[1] level in SG 1-2 to decrease very slow!". Control of SG 1-2 level was obtained at
0517 hours on 3/25/78 by manually closing the motor operated Main Feedwater Block
Valve FW 779 which had not been completely closed by its motor operator. Adjustments
were then made to the motor operator of the valve .
THER STATUS OF DISCOVERY DESCRIPTION (32) STATUS POWER OTHER STATUS (30) METHOD OF DISCOVERY DESCRIPTION (32) A (31) Operator Observation of SG level
1 5 2 28 10 12 13 44 45 46
1 6 Z 33 Z 34 NA NA 45 NA
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39 1 7 0 0 0 37 Z 38 NA
7 8 9 PERSONNEL INJURIES NUMBER DESCRIPTION 41
50 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0
100 100 100 100 100 100 100 100 100 100
NRC USE CIVEY
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James Marley PHONE 419-259-3000, Ext. 275

TOLEDO EDISON COMPANY DAVIS-BESSE UNIT ONE NUCLEAR POWER STATION SUPPLEMENTAL INFORMATION FOR LER NP-33-78-36

DATE OF EVENT: March 25, 1978

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Steam Generator 1-2 level exceeded 86%

Conditions Prior to Occurrence: The unit was in Mode 2, with Power (MWT) = 0, and Load (MWE) = 0.

Description of Occurrence: While performing a reactor shutdown to Mode 3 for maintenance work, Steam Generator 1-2 (SG 1-2) level was raised to between 35% and 70% on the operating range per SP 1102.10, "Station Cooldown and Shutdown Procedure". Reactor power was reduced to 0%. SG 1-2 level continued to increase even though the Startup Feedwater Valve SP7A was closed. An operator then closed the Startup Feedwater Isolation Valve FW 162. The level in SG 1-2 continued to increase. At 0455 hours on March 25, 1978, SG 1-2 level exceeded the 86% maximum limit and increased to a level of 88%, which placed the unit in Action Statement B of Technical Specification 3.4.5.

At 0500 hours, an operator entered containment and opened one drain on each Steam Generator. The level in SG 1-2 decreased very slowly until the operator manually closed the Main Feedwater Block Valve FW 779, which had not closed tightly even though Control Room indication showed a green light indicating FW 779 was closed. After manually fully closing FW 779, SG 1-2 level decreased rapidly. At 0517 hours on March 25, 1978, SG 1-2 level was less than the 86% limit on Steam Generator Feedwater Level. On the operating range instrumentation, 86% corresponds to 348 inches.

Designation of Apparent Cause of Occurrence: The cause for the SG 1-2 level to exceed 86% was the failure of FW 779's motor operator to completely close the valve causing the overfilling of SG 1-2.

Analysis of Occurrence: There was no danger to the health and safety of the public or unit personnel. No steam line break occurred during the short period of time the Steam Generator level slightly exceeded 86%.

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Corrective Action: The operator closed the Startup Feedwater Isolation Valve; he entered containment and opened a drain on SG 1-2. Both actions had relatively little effect on SG 1-2's level. The operator manually closed FW 779 tightly causing the level to drop quickly. At 0517 hours on March 25, 1978, SG 1-2 level was less than 86% which removed the unit from Action Statement B of Technical Specification 3.4.5. A Work Request, MWO 78-740, was written and under Maintenance Work Order 78-191, Maintenance investigated the problem with FW 779. Adjustments were made to the number two rotor contact in the valves motor operator.

Failure Data: This is not a repetitive occurrence. One prior occurrence of Steam Generator level exceeding 348 inches was due to personnel error while preparing for a turbine roll (see Licensee Event Report NP-33-77-30).

LER #78-033