### U.S. NUCLEAR REGULATORY COMMISSION OFFICE OF INSPECTION AND ENFORCEMENT

RECION III

Report No. 50-346/77-19

Docket No. 50-346

License No. CPPR-80

Toledo Edison Company Licensee: Edison Plaza 300 Madison Avenue Toledo, OH 43652

Facility Name: Davis-Besse Nuclear Power Station, Unit 1 Inspection At: Davis-Besse Site, Oak Harbor, OH Inspection Conducted: April 26-28; May 3, and 4, 1977

Inspectors: C. C. Williams

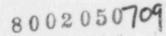
Approved By; D. W. Hayes, Chief

7/13/77 date signed

Inspection Summary

Inspection on April 26-28; May 3, and 4, 1977, (Report No. 50-346/77-19) Areas Inspected: (1) Staking of stem lock nuts on limitorque valve operators; (2) high pressure injection pump modification; (3) large and small piping hangars and anchors; (4) system interaction considerations (EIR's); (5) HVAC system rework activities; (6) rework of small Class 1 and 2 valves; (7) status of penetration sealing activity; and (8) modifications relative to control power circuits for primary pump feed breakers. Included in these activities was the review and resolution of a previously identified unresolved matter and a deficiency reported pursuant to 10 CFR Part 50.55(e). This inspection involved a total of 49 inspector-hours by two inspectors.

Results: No items of noncompliance, deviations or unresolved matters were identified.



### DETAILS

#### Persons Contacted

#### Principal Licensee Employees

\*L. E. Roe, Vice President
\*J. D. Lenardson, QA Manager
\*R. E. Blanchon, Construction Supervisor
\*G. E. Eichenauer, QA Engineer
E. M. Wilcox, QA Engineer
\*C. R. Domec, Nuclear Engineer
\*E. C. Novak, General Superintendent
D. Poage, QA Engineer
D. Dibert, Operations Engineer

### Other Personnel

C. L. Houston, Field Construction Manager (Bechtel) W. C. Lowery, QA Engineer (Bechtel) J. Faye, Project Engineer (Bechtel) J. D. Heaton, Quality Assurance Engineer (Bechtel) A. Casalena, QC Inspector (Bechtel) E. D. Doer, State Inspector P. McDonal, Project Manager (Lum-IRSAY) L. Lensen, Instrument Engineer (Bechtel)

The inspector also contacted and interviewed other licensee, Bechtel and contractor employees, including members of the Quality Technical and Engineering staff.

\*denotes those attending the exit interview (April 28 and May 4, 1977).

Licensee Action on Previous Inspection Findings

None identified.

#### Functional or Program Areas Inspected

 Limitorque Operated Valves - Type SMB - Staking of Stem Lock Nut (50-346/77-18)

This matter was originally reported by the licensee pursuant to 10 CFR Part 50.55(e). Satisfactory completion of the work and verification by the NRC inspector was made a condition of the







operating license issued April 22, 1977. (Item No. C.5, Attachment No. 2 of License NPF-3). During this inspection the corrective action identified in the licensee's final report, dated April 27, 1977, was examined. The inspector verified, through component observation, review of records, and discussion with licensee personnel, that all (232) limitorque operated valves were inspected relative to the required staking of the stem lock nuts. One hundred forty-four of the 232 were in safety related systems. Lock nuts on 65 of the 144 safety related valves required rework (staking). The inspector ramdonly selected a sample of 20 for examination and record review. No discrepancies were noted.

This matter is closed.

## 2. High Pressure Injection Pump Modification (50-346/77-18)

As previously reported, documentation was inadequate to establish that the modification of the subject pumps was in accordance with FSAR and specification requirements. Satisfactory resolution of this matter was made a condition of the plant operating license (Item C.1, Attachment 2). During this inspection, the inspector established, through document review and discussion with licensee personnel, that the final documentation adequately assures conformance to the requirements.

This previously identified unresolved matter is closed.

# 3. Large and Small Pipe Hangars and Anchors (50-346/77-18)

Satisfactory resolution of this issue and varification by the NRC inspector was made a condition of the plant operation license (Items C.3 and C.4, Attachment 2). During this inspection, the inspector examined that portion of this work remaining to be completed. While no significant deficiencies were noted, this work remains incomplete. The major portion of the remaining work involves completion of the engineering and quality control documentation. This overall effort involves two site contractors, ITT Grinnell and Babcock and Wilcox. The inspector examined six completed hangars and found no discrepancies.

This matter remains unresolved and will be examined during subsequent inspections.

System Interaction - EIR's 5000 and 8000 (50-346/77-15)

As previously reported, this reinspection and rework activity was incomplete. The incomplete issues were divided into two separate areas of consideration relative to their need to be completed for various modes of plant operation.







 One hundred three of these EIR's (Engineering Inspection Report) required completion prior to mode 5, plant operation (cold shutdown), and were made a condition of the licensee. (ltems B.2(a), (b), (c), and (d), Attachment 2)

During this inspection, the inspector verified, through walkdown inspection, observation and examination of documentation, and discussion with licensee personnel, that each of these items were adequately resolved.

Examples of specific items included in the inspection effort are as follows:

(1) EIR No. 5585/Closure Letter No. BT 7104

Four Items Samples:

Equipment No. 37126A Equipment No. 37487A Equipment No. 38709B Equipment No. 37126A-2

Each of the above items contain an average of 12 elements (i.e., conduits, etc.) that required inspection and observation by the inspector.

(2) EIR No. 81831

Three Items Sampled:

Equipment No. 36606A Equipment No. 36840A Equipment No. 36605A

Each of the above items had an average of six elements that required inspection and observation by the inspector.

b. About 47 of the subject EIR's require completion prior to mode 4, plant operation (hot shutdown), and were also made a condition of the licensee (Items C.7.(a) and (b), Attachment 2)

During this inspection, the licensee reported that these items, required for mode 4, have not been completed and that many were awaiting an engineering evaluation.

These matters remain open and will be reviewed during a subsequent inspection.





## 5. Heating Ventilation and Air Conditioning System (50-346/77-15)

Satisfactory resolution of this matter was made a condition of the plant operating license, (Item C.2, Attachment 2). As previously reported, the reinspection and subsequent repair effort, relative to welding (seismic considerations), remained to be completed. At this time, Lum-IRSAY (site contractor) has completed all reinspection and rework activity. The only aspect of this rework remaining, is the completion and turnover of the final documentation. Examination of the in process documentation and its control, did not disclose any discrepancies.

This matter remains unresolved and will be reviewed during subsequent inspections.

### 6. Reworked Small Class 1, 2, and 3 Valves (50-346/77-18)

This item remains unresolved. It's satisfactory completion and varification by NRC inspection was made a condition of the plant operating license (Item C.1, Attachment 2). One of the five previously reported valves was determined through examination to be complete with final documentation available. The other four valves (CS-22, CS-22, DA-38 and CV-117) remain to be completed.

This matter will be reviewed during the next NRC inspection.

### 7. Penetration and Blockout Sealant - Bisco (50-346/77-18)

The inspector reviewed the status of completion of this work. At this time the licensee estimates that, including the balance of the plant work, the sealant installation work is about 50% completed.

Observation of work activity and previous examinations of the document controls indicated that this work is progressing in accordance with the requirements of the specification and the TECo QA Manual.

This matter remains open and further review is planned during future inspections.

## Control Power Circuit Modifications - Primary Pump Breakers Safety Evaluation Report Supplement No. 2, Item No. 7.10)

The RIII inspector verified that the DC control power for both the 13.8KV backup and primary for the reactor coolant pump motors has been divided into two sections, i.e., the back-up and primary breakers are supplied control power from different sources. The primary breakers are supplied from DC distribution panels "DAP" and "DBP". The back-up breakers are supplied from DC distribution panels "DAN" and "DBN".





All work was performed in accordance with System Revision Notice No. 257E, and Construction Work Permits No. 3-E-7 and No. 3-E-8.

By action of a hand operated DC control power transfer switch, both the primary and back-up breakers (for maintenance considerations) can be supplied from the same DC distribution panel. This could compromise the newly installed safety feature if the switch positions are not properly controlled. Administrative provisions are being established to control inadvertant switch operation. System Procesure (SP 1107.04 titled, "13.8KV Buses Switching Procedure" is being revised by the licensee to include the above considerations.

This matter is closed.

