

50-346

## NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

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TO: Mr Rusche

FROM: Toledo Edison Co  
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## DESCRIPTION

Ltr trans the following:

ACKNOWLEDGED

DO NOT REMOVE

PLANT NAME: Davis Besse #1

## ENCLOSURE

Containment Vessel Integrated Rate Test Rpt...  
.....(3 cys encl rec'd, one cy placed in Reg  
File & 2 cys sent to LPM Engle for Dist, all  
others on Dist ltr only)

## SAFETY

## FOR ACTION/INFORMATION

ENVIRO

12-27-76

ehf

ASSIGNED AD:

Vassallo (Ltr)

ASSIGNED AD:

BRANCH CHIEF:

Stolz (Ltr)

BRANCH CHIEF:

PROJECT MANAGER:

Engle w/2 encls

PROJECT MANAGER:

LIC. ASST. :

Hyton (Ltr)

LIC. ASST. :

## INTERNAL DISTRIBUTION

REG FILE w/encl

SYSTEMS SAFETY

PLANT SYSTEMS

SITE SAFETY &amp;

NRC PDR Ltr

HEINEMAN

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ENVIRO ANALYSIS

I &amp; E

SCHROEDER

BENAROYA

DENTON &amp; MULLER

OELD

LAINAS

GOSSICK &amp; STAFF

ENGINEERING

IPPOLITO

ENVIRO TECH.

MIPC

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CASE

KNIGHT

BALLARD

HANAUER

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HARLESS

PAWLICKI

STELLO

SITE TECH.

PROJECT MANAGEMENT

REACTOR SAFETY

OPERATING TECH.

GAMMILL

BOYD

ROSS

EISENHUT

STAPP

P. COLLINS

NOVAK

SHAO

HULMAN

HOUSTON

ROSZTOCZY

BAER

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CHECK

BUTLER

SITE ANALYSIS

MELTZ

GRIMES

VOLLMER

HEITEMES

AT &amp; I

BUNCH

SKOVHOLT

SALTZMAN

J. COLLINS

RUTBERG

KREGER

## EXTERNAL DISTRIBUTION

## CONTROL NUMBER

LPDR: A/C in for PA (Ltr)

NAT. LAB:

BROOKHAVEN NAT. LAB.

TIC:

REG V. IE

ULRIKSON (ORNL)

NSIC:

LA PDR

ASLB:

CONSULTANTS:

ACRS CYS HOLDING/SEPT

12908

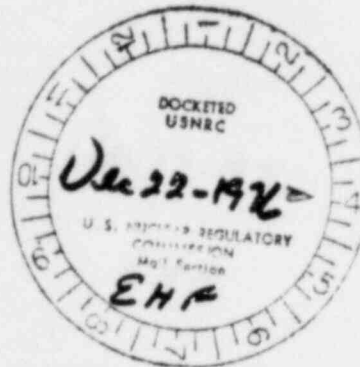
# REGULATORY DOCKET FILE COPY



Docket No. 50-346

December 22, 1976

Serial No. 169



LOWELL E. ROE  
Vice President  
Facilities Development  
(419) 259-5242

Mr. Benard C. Rusche, Director  
Office of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. Rusche:

50-346

Attached please find three copies of the "Containment Vessel Integrated Rate Test" report for the Davis-Besse Nuclear Power Station Unit No. 1, which is being submitted to the Nuclear Regulatory Commission in accordance with Appendix J of 10CFR Part 50. The results of the test show that the actual measured containment vessel leakage, when the containment vessel is pressurized to the peak calculated accident pressure, is less than 75% of the maximum allowable leakage assumed in safety analyses. The containment vessel has therefore been assessed as being fully capable of performing its intended function in the operation of the Davis-Besse Nuclear Power Station Unit No. 1.

Yours very truly,

Attachment: Davis-Besse Nuclear Power Station Unit No. 1 Containment Vessel Integrated Leak Rate Test

bj b/5

cc: w/l  
James G. Keppler  
Regional Director, Region III  
Directorate of Regulatory Operations  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

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