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Docket No. 50-269

Morris Rosen, Technical Assistant to Deputy Director for Reactor Projects, L THRU: A. Schwencer, Chief, Pressurized Water Reactors Branch Noordenal Schwencer

SUMMARY OF ACRS MEETING - OCONEE NUCLEAR POWER STATION UNIT 1 - OPERATING LICENSE APPLICATION

Representatives of the Staff met with the ACRS on January 12, 1973 concerning the status of the Oconee Nuclear Power Station Unit 1.

Items Discussed During the ACRS/Staff Session

General Items

- 1. We informed the Committee that the applicant expects to be ready to load fuel in the first week in February and that there is a reasonable probability that we can resolve the outstanding matters with the applicant by this time. We indicated, therefore, that issuance of the license might well occur before the February meeting of the ACRS. We noted that a future supplement to the SER would address all the issues. The Staff's intent is to issue this supplement prior to Committee review of the remaining Units 2 and 3.
- The Committee requested that it be kept informed on our plans for site visits in the near future since the subcommittee is considering another visit to the Oconee site.
- '3. A member of the Committee raised a question as to whether the Committee should document their review of the additional information prior to licensing Unit 1. Although the question appeared to be addressed to other Committee members, Mr. Boyd said that the Staff noted this concern and would take it into consideration.

Discussion

A. The project manager made a presentation on the status of the reactor internals redesign and modification and the resultant Technical Review evaluation. During the course of the presentation the Committee raised

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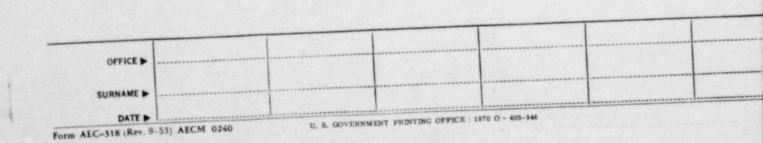
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some questions which could only be answered in a general or qualified manner since the appropriate Technical Review representatives were unavailable for the meeting. The majority of questions raised appeared to have been answered satisfactorily.

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The following are the major areas of concern which could be addressed by the Staff at the meeting in only a general manner:

- 1. One member of the Committee asked if changes in flow conditions after the core is installed could introduce adverse effects not present during the preoperational vibration tests. The Staff noted that flow would be reduced but did not speculate on the consequences. The Staff pointed out that the applicant plans to continue to monitor for loose parts by external electronic listening" systems during reactor operations. One member of the Committee appeared to be interested in means to assure that flow changes would not result in adverse effects.
- 2. Another member of the Committee asked how B&W had calculated vortex shedding frequencies before and after modification and on what basis did it conclude that failure induced by vortex shedding was possible originally and now eliminated. The Staff stated that vortex shedding is calculated by a "textbook" formula and that it was from additional tests in the "1/6 scale" model tests that B&W concluded that local flow velocities could put the vortex shedding frequencies in the critical range for the original internals (in the range of the natural frequencies of the components). By shifting the natural frequencies of the new components by design B&W separated the natural frequencies from the calculated vortex shedding frequencies.
 - 3. One question was what would have happened if the core had been in place at the time the internals failed. The Staff did not speculate on the potential damage but anticipated that the loose parts monitoring system would detect the failure.
 - 4. The Staff verified for the Committee that the 5% power restriction, to be in effect until the Staff is satisfied with the adequacy of the preoperational vibration tests, would permit the reactor to be operated at full flow.



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- 5. The Staff presented a table which thowed that the measured stresses in the redesigned internals were well below the allowable stresses. The Committee asked if B&W had compared these values against previous test data from the original internals. The Staff was not prepared to answer this question because the original internals were not instrumented in the same manner or to the same degree.
- B. The project manager made a presentation on the status of the flow limiters which will be installed in the Oconee Unit 1 flood lines.
 - 1. The Committee's main concerned appeared to be whether or not the restrictors would stay in place during a LOCA after experiencing inservice environment for many years (chemical attack etc.). The Staff pointed out that it has not completed its review of the structural aspects of this design but that it is not unlike the thermal sleeve situation. Thermal sleeves have been approved for plants in operation.
- C. The project manager made a presentation on the status of the Staff's steamline break review. Preliminary information had been provided by the Staff 'o the ACRS but had not reached the individual Committee members by the time of the meeting. The Committee had no questions.

Original Signed by Irving A. Peltier

I. A. Peltier, Project Manager Pressurized Water Reactors Branch No. 4 Directorate of Licensing

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