MONTHLY NRC

SUMMARY OF OPERATING EXPERIENCE,

CHANGES, TESTS, AND EXPERIMENTS

PER REGULATORY GUIDE 1.16 AND 10 CFR 50.59

FOR

DRESDEN NUCLEAR POWER STATION

COMMONWEALTH EDISON COMPANY

UNIT	DOCKET	LICENSE
1	050-010	DPR-2
2	050-237	DPR-19
3	050-249	DPR-25

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1.0 Introduction

Dresden Nuclear Power Station is a three reactor generating facility owned and operated by the Commonwealth Edison Company of Chicago, Illinois. Dresden Station is located at the confluence of the Kankakee and Des Plaines Rivers, in Grundy County, near Morris, Illinois.

Dresden Unit 1 is a General Electric Boiling Water Reactor with a design net electrical output rating of 200 megawatts electrical (MWe). The unit is retired in place with all nuclear fuel removed from the reactor vessel. Therefore, no Unit 1 operating data are provided in this report.

Dresden Units 2 and 3 are General Electric Boiling Water Reactors with design net electrical output ratings of 794 MWe each.

Waste heat is rejected to a man-made cooling lake using the Kankakee River for make-up and the Illinois River for blowdown.

The Architect-Engineer for Dresden Units 2 and 3 was Sargent and Lundy of Chicago, Illinois.

This report was compiled by Gerrine Paramore of the Dresden Technical Staff, telephone number (815)942-2920 extension 2364.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR OCTOBER, 1989

2.1 UNIT 2 MONTHLY OPERATING EXPERIENCE SUMMARY

10-01-89 to 10-22-89	Unit 2 entered the month on line and operating at approximately 716 MWe.
10-23-89 to 10-30-89	At approximately 2100 hours on October 23, 1989, the Unit 2 High Pressure Coolant Injection (HPCI) System was declared inoperable due to concerns of potential steaming in the discharge piping. The elevated discharge piping temperatures were caused by backleakage from the reactor feedwater system. The HPCI system was isolated until further review allowed for a modified valve line up to permit continuous operation. A seven-day Limiting Condition of Operation (LCO) was started at 2100 hours in accordance with Technical Specification 3.5.C.
10-31-89	At approximately 0800 hours on October 31, 1989, an Unusual Event was declared on Dresden Unit 2 and a unit shutdown was initiated pending completion of HPCI operability testing. The load reduction was secured upon return to service and testing of the HPCI System. HPCI was declared operable and the Unusual Event was terminated at 1026 hours on October 31, 1989.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR OCTOBER, 1989

2.2 UNIT 3 MONTHLY OPERATING EXPERIENCE SUMMARY

10-01-89 to 10-31-89	Unit 3 entered the month on line and operating at approximately 811 MWe. The unit operated in Economic Generation Control or at loads requested by the System Load Dispatcher for the remainder of the month.

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3.1 OPERATING DATA REPORT - UNIT TWO

DOCKET NO. 050-237
UNIT DRESDEN TWO
DATE: NOVEMBER 1, 1989
COMPLETED BY: G.M. PARAMORE
TELEPHONE (815) 942-2920

OPERATING STATUS

1. REPORTING PERIOD OCTOBER 1989 — GROSS HOURS IN REPORTING PERIOD 745
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt) 2,527 — MAX DEPEND CAPACITY (MWe-Net) 772; DESIGN ELECTRICAL RATING (MWe-Net) 794
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net) — N/A
4. REASONS FOR RESTRICTION (IF ANY)

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	745.0	6,013.9	129,559.8
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
7. TIME GENERATOR ON-LINE (HOURS)	745.0	5,901.9	123,927.3
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWHt-Gross)	1,819,952	12,988,734	255,480,596
10. ELECTRICAL ENERGY GENERATED (MWHe-Gross)	584,774	4,154,670	81,639,403
11. ELECTRICAL ENERGY GENERATED (MWHe-Net)	557,298	3,945,964	77,188,870
12. REACTOR SERVICE FACTOR (%)	100.0	82.4	75.9
13. REACTOR AVAILABILITY FACTOR (%)	100.0	82.4	75.9
14. SERVICE FACTOR (%)	100.0	80.9	72.6
15. AVAILABILITY FACTOR	100.0	80.9	72.6
16. CAPACITY FACTOR (USING MDC) (%)	96.9	70.1	58.6
17. CAPACITY FACTOR (USING DESIGN MWe) (%)	94.2	68.1	57.0
18. FORCED OUTAGE FACTOR (%)	0.0	2.1	10.9

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS
(TYPE DATE AND DURATION OF EACH)

125VDC TEST OUTAGE, 12-13-89, 13 DAYS

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP

N/A

3.2 OPERATING DATA REPORT - UNIT THREE

DOCKET NO. 050-249
UNIT DRESDEN THREE
DATE: NOVEMBER 1, 1989
COMPLETED BY: G.M. PARAMORE
TELEPHONE (815) 942-2920

OPERATING STATUS

1. REPORTING PERIOD OCTOBER 1989 — GROSS HOURS IN REPORTING PERIOD 745

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2,527 — MAX DEPEND CAPACITY (MWe-Net) 773
DESIGN ELECTRICAL RATING (MWe-Net) 794

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net) N/A

4. REASONS FOR RESTRICTION (IF ANY)

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	745.0	6,552.7	119,461.0
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
7. TIME GENERATOR ON-LINE (HOURS)	745.0	6,466.2	111,590.3
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWHt-Gross)	1,785,524	14,968,287	229,952,746
10. ELECTRICAL ENERGY GENERATED (MWHe-Gross)	574,306	4,831,514	74,220,223
11. ELECTRICAL ENERGY GENERATED (MWHe-Net)	184,754	4,243,598	69,981,024
12. REACTOR SERVICE FACTOR (%)	100.0	89.8	74.5
13. REACTOR AVAILABILITY FACTOR (%)	100.0	89.8	74.5
14. SERVICE FACTOR (%)	100.0	88.6	69.6
15. AVAILABILITY FACTOR	100.0	88.6	69.6
16. CAPACITY FACTOR (USING MDC) (%)	95.3	81.7	56.8
17. CAPACITY FACTOR (USING DESIGN MWe) (%)	92.8	79.5	55.3
18. FORCED OUTAGE FACTOR (%)	0.0	3.4	12.2

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS
(TYPE DATE AND DURATION OF EACH)

REFUEL OUTAGE, 12-3-89, 10 WEEKS

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP

N/A

3.3 AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-237

UNIT II

DATE NOVEMBER 1, 1989

COMPLETED BY G. PARAMORE

TELEPHONE 815/942-2920

MONTH OCTOBER, 1989

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	724	17	771
2	769	18	776
3	769	19	779
4	772	20	782
5	771	21	785
6	772	22	754
7	773	23	756
8	754	24	768
9	776	25	766
10	773	26	763
11	767	27	756
12	773	28	622
13	771	29	585
14	753	30	683
15	709	31	689
16	760		

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3.4 AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-249

UNIT III

DATE NOVEMBER 1, 1989

COMPLETED BY G.P. PARAMORE

TELEPHONE 815/942-2920

MONTH OCTOBER, 1989

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	769	17	752
2	770	18	751
3	769	19	600
4	603	20	766
5	751	21	719
6	750	22	727
7	745	23	722
8	770	24	710
9	768	25	720
10	764	26	737
11	772	27	726
12	768	28	728
13	760	29	758
14	739	30	725
15	752	31	724
16	745		

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DOCKET NO. 050-237
UNIT NAME DRESDEN UNIT II
DATE November 1, 1989
COMPLETED BY G. Paramore
TELEPHONE (815)942-2920

3.5 UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OCTOBER, 1989

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR[3]	LICENSEE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
7	10/31/89	F	2:26	H	5	89-29/050237	BJ	FCV	At 0800 hours on October 21, 1989, an Unusual Event was declared on Dresden Unit 2 and a unit shutdown was initiated due to the HPCI System being inoperable for seven days. A load reduction was secured upon return to service and performance of Dresden Operating Surveillance (DOS) 2300-1 testing of the HPCI System. HPCI was declared operable and the Unusual Event terminated at 1026 hours on October 31, 1989. Refer to Secton 2.1.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensee Examination
F-Administrative
G-Operational Error
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)
5-Load Reduction

4
Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit I - Same Source

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DOCKET NO. 050-249
UNIT NAME DRESDEN UNIT III
DATE November 1, 1989
COMPLETED BY G. Paramore
TELEPHONE (815)942-2920

3.6 UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OCTOBER, 1989

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR[3]	LICENSEE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
5	None	-	-	-	-	-	-	-	-

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensee Examination
F-Administrative
G-Operational Error
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)
5-Load Reduction

4
Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit I - Same Source

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3.7 STATION MAXIMUM DAILY ELECTRICAL LOAD DATA
DRESDEN STATION
OCTOBER, 1989

DAY	HOUR ENDING	MAXIMUM DAILY LOAD KW
1	2300	1,606,400
2	0900	1,609,000
3	1100	1,614,500
4	2400	1,518,100
5	1200	1,608,900
6	2100	1,611,300
7	1800	1,613,700
8	1900	1,616,800
9	2400	1,613,600
10	0200	1,614,500
11	0600	1,607,700
12	1400	1,612,200
13	0200	1,606,900
14	0100	1,578,100
15	0100	1,548,500
16	1500	1,582,100
17	1900	1,596,800
18	1800	1,604,000
19	2400	1,559,500
20	1800	1,630,400
21	1600	1,586,100
22	2100	1,574,500
23	0100	1,572,700
24	0100	1,557,000
25	1200	1,587,200
26	0100	1,576,200
27	0400	1,555,100
28	1100	1,543,400
29	2400	1,438,300
30	1500	1,501,400
31	2400	1,551,700
TOTAL		48,996,600

4.0 UNIQUE REPORTING REQUIREMENTS

4.1 MAIN STEAM RELIEF VALVE OPERATIONS

Relief valve operations during the reporting period, October, 1989, are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit	Date	Valves Actuated	No. and Type of Actuations	Plant Conditions	Description of Events
2/3	10/89	Valve Serial No's: BK 7052 BK 7080	2, Bench Tested		These Electromatic Relief Valves were spares that were bench tested.

4.2 OFF-SITE DOSE CALCULATION MANUAL CHANGES

There were no changes to the Off-Site Dose Calculation Manual during October, 1989.

4.3 MAJOR CHANGES TO THE RADIOACTIVE WASTE TREATMENT SYSTEMS

There were no major changes to the radioactive waste treatment systems at Dresden during October, 1989.

4.4 FAILED FUEL ELEMENT INDICATIONS

4.4.1 Unit 2

Dresden Unit 2 fuel performance during October 1989 continued to show no indications of leaking fuel. This is based on the sum of the activities of the six noble gases as measured at the recombiner. Based on the reported data, Unit 2 had acceptable fuel performance.

4.4.2 Unit 3

Dresden Unit 3 fuel performance during October 1989 continued to show no indications of leaking fuel. This is based on the sum of the activities of the six noble gases as measured at the recombiner. Based on the reported data, Unit 3 had acceptable fuel performance.

5.0 PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

5.1 Amendments to Facility License or Technical Specifications

The license amendments and/or Technical Specification changes which were approved and implemented for use during the reporting period are listed below.

5.1.1 Unit 2

None

5.1.2 Unit 3

None

5.2 Changes to Procedures Which are Described in the FSAR (Units 2 and 3)

Table 5.2.1, attached, summarizes the revisions to procedures described in the FSAR which were approved during thereporting period.

TABLE 5.2.1

CHANGES TO PROCEDURES WHICH ARE DESCRIBED IN THE FSAR (UNITS 2 AND 3)

PROCEDURE TYPE	PROCEDURE NO.	PROCEDURE TITLE/DESCRIPTION	SUMMARY OF CHANGES
Dresden Administrative Procedure (DAP)	DAP 2-3	Operation and Control of the Central and Satellite Files	4
Dresden Operating Surveillance (DOS)	DOS 500-1	Unit 2(3) Manual Scram Circuit Sensor Tests	2

NOTES: 1. Administrative change; intent of procedure unchanged.
2. Changed for clarification, intent of procedure unchanged.
3. Changed to incorporate requirements for new equipment; intent of procedure unchanged
4. Changed to implement improved testing/calibration methodology; intent of procedure unchanged.

5.3 Significant Tests and Experiments Not Described in the FSAR (UNITS 2 and 3)

Significant special procedures involving tests not described in the FSAR which were approved during the month are listed below.

Procedure No.	Procedure Title/Description
SP 89-9-83	Inspection and maintenance of GE 4 kilovolt (kV) breakers.
SP 89-9-86	Unit 2 and 3 feedwater flow rate determination. Results will be analyzed and used for in-place calibration of the feedwater flow nozzles.
SP 89-10-88	Unit 3 Drywell Floor Drain Sump Pump A Run Time Modification Test.
SP 89-10-112	This procedure was utilized to determine if high stall flows were caused by a leaking Control Rod Drive/Hydraulic Control Unit valve 3-0305-121 or if it was internal to the drive. Test was used to determine the need for rebuilding of the CRD during the upcoming Unit 3 Refuel Outage.

5.4 Safety related maintenance (Units 2 and 3)

Safety related maintenance activities are summarized in the attached tables.

DRESDEN UNIT 2
5.4 SAFETY RELATED MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION CAUSE	MALFUNCTION RESULT	CORRECTIVE ACTION
MOV-2-3703	PREVENTIVE	N/A	----------	----------	MOUNTED VOTES SENSOR ON VALVE YOKE AND TESTED VALVE
2-6660 U2 D/G TURBO OIL CIRCULATING PUMP MOTOR	PREVENTIVE	N/A	----------	----------	REPAIRED POWER FEED CONNECTION ON U2 DIESEL GENERATOR
2-MO-1001-2C	PREVENTIVE WR D72206	N/A	----------	----------	REBUILT LIMITORQUE, CHANGED WORM BEARINGS, GASKETS AND SEALS
LPRM REPLACEMENT	PREVENTIVE WR D77549	N/A	----------	----------	REPLACED LPRM'S
MOV 2-1001-5A	PREVENTIVE WR D79002	N/A	----------	----------	MOUNTED VOTES SENSOR ON VALVE YOKE AND TESTED VALVE
MOV 2-1501-5D	PREVENTIVE WR D79015	N/A	----------	----------	MOUNTED VOTES SENSOR ON VALVE YOKE AND TESTED VALVE
2-3706 VALVE	CORRECTIVE WR D80544	N/A	----------	----------	CLEANED THREADS AND ADJUSTED PACKING TO ELIMINATE LEAKAGE
DIESEL GENERATOR	CORRECTIVE WR D80875	N/A	----------	----------	CLEANED JOINT AND INSTALLED NEW SEAL AND WATER INLET PIPE GASKET
U2 DIESEL GENERATOR	CORRECTIVE WR D81519	N/A	----------	----------	CLEANED ALL THREADS AND APPLIED PST
A02-1601-63 DRYWELL VENT BYPASS	CORRECTIVE WR D81875	N/A	----------	----------	INSTALLED NEW PRESSURE SWITCH AND LOWERED SETPOINT

DRESDEN UNIT 2
SAFETY RELATED MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION CAUSE	MALFUNCTION RESULT	CORRECTIVE ACTION
MOV 1001-2C	PREVENTIVE WR 083900	N/A	----------	----------	REPLACED SEALTITE, BRIDGED AND MEGGERED
SPARE 4KV CB	PREVENTIVE WR 083920	N/A	----------	----------	REPLACED BROKEN PLATE INSULATOR
HPCI ROOM COOLER 2-5747	CORRECTIVE WR 084443	N/A	----------	----------	ADDED 2ND BEARING TO SHAFT AND CLEANED AREA
CEILING ABOVE 2A RBCCW PUMP SUCTION VALVE	CORRECTIVE WR 085005	N/A	----------	----------	FABRICATED AND INSTALLED 5 PAD EYES FOR RAISING AND LOWERING HOT TOP EQUIPMENT
2A H202 CAM	PREVENTIVE WR 085553	N/A	----------	----------	WROTE NEW WORK REQUESTS TO REPLACE SAMPLE PUMP AND REPAIR CELL FLOW PROBLEMS
4KV BREAKER SERIAL# 0204A2317-004	PREVENTIVE WR 085687	N/A	----------	----------	PERFORMED PREVENTIVE MAINTENANCE AND REPLACED SPRING CHARGING MOTOR
2-2402A D2 A H202 MONITOR SAMPLE PUMP	CORRECTIVE WR 085718	N/A	----------	----------	INSTALLED NEW PUMP
LPRM 3B-08-33	CORRECTIVE WR 085960	N/A	----------	----------	STRAIGHTENED PEN AND TESTED
2-8350-2 U2 250VDC BATTERY CHARGER	PREVENTIVE WR 085987	N/A	----------	----------	RESET VOLTAGES IN FLOAT AND EQUALIZE
LPRM 40-33C	PREVENTIVE WR 086078	N/A	----------	----------	PERFORMED PLATEAU, FOUND NO PROBLEMS

DRESDEN UNIT 2
SAFETY RELATED MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION CAUSE	MALFUNCTION RESULT	CORRECTIVE ACTION
2-6723-21 4KV CIRCUIT BREAKER SER# 0204A2317-003	PREVENTIVE WR D86197	N/A	---	---	CYCLED BREAKER
'A' FUEL POOL RAD MONITOR	CORRECTIVE WR D86650	N/A	---	---	REPLACED CABLE AND SENSOR AND CALIBRATED
2-2452A D2 'A' H202 MONITOR PUMP	CORRECTIVE WR D86718	N/A	---	---	INSTALLED NEW ANALYZER PUMP
1501-21 LPCI INJECTION VALVE	PREVENTIVE WR D86732	N/A	---	---	PERFORMED SIGNATURE
M02-1501-21B	PREVENTIVE WR D86740	N/A	---	---	PERFORMED SIGNATURE
2-5850 RB/TB INTERLOCK DOOR	CORRECTIVE WR D87022	N/A	---	---	ADJUSTED DOOR SPEED, BACKCHECK, SWEEP AND LATCH
2-700-RBM #8	CORRECTIVE WR D87033	N/A	---	---	REPLACED CARD AND COMPLETED PROCEDURE
'C' MSL RAD MONITOR	CORRECTIVE WR D87090	N/A	---	---	REPLACED MONITOR

DRESDEN NIT 3
SAFETY RELATED MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION CAUSE	MALFUNCTION RESULT	CORRECTIVE ACTION
U3 3-203-3A ROCK TARGET	PREVENTIVE WR D66516	N/A	---	---	INSTALLED NEW SOLENOID
3-2330-129A HPCI HFA RELAY	PREVENTIVE WR D81037	N/A	---	---	REPLACED RELAY
3-1502-3C LPCI PUMP RUNNING ALARM	CORRECTIVE WR D82684	N/A	---	---	INSTALLED SWITCHES IN CUBICLE FOR 4KV BREAKER
3-2452B D3 H2O2 ANALYZER 'B' FOR PRIMARY CONTAINMENT	PREVENTIVE WR D84445	N/A	---	---	PERFORMED 18 MONTH INSPECTION
D3 D/W PERSONNEL HATCH	PREVENTIVE WR D85221	N/A	---	---	INSTALLED AND REMOVED STRONGBACKS FOR LLRT
3-2452B D3 'B' DW/TORUS H2O2 SAMPLE PUMP	CORRECTIVE WR D85282	N/A	---	---	REPLACED PUMP
MO3-1501-5B VALVE 3B LPCI SUCTION VALVE	PREVENTIVE WR D85660	N/A	---	---	REPLACED MOTOR GASKET
MOV 3-1001-2B	PREVENTIVE WR D86074	N/A	---	---	PERFORMED ELECTRICAL P.M. AND INSPECTION PER PROCEDURE, BRIDGED AND MEGGERED MOTOR AND PERFORMED SIGNATURE, REPLACED PARTS AS FOUND NECESSARY
3-263-106B U3 'B' FUEL ZONE REACTOR WATER LEVEL INDICATOR	CORRECTIVE WR D86984	N/A	---	---	REPLACED TRANSMITTER

DRESDEN UNIT 2/3
SAFETY RELATED MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION CAUSE	MALFUNCTION RESULT	CORRECTIVE ACTION
SPARE 18" 1200 PRATT VALVE FOR TORUS OR DRYWELL VENT	PREVENTIVE WR D81888	N/A	---	---	REBUILT SPARE VALVE

5.5 Completed Safety Related Modifications (Units 2 and 3)

Unit 2 and Unit 3 safety related modification packages closed during the month of October, 1989 are listed below. Only modifications which have been completely closed are listed; modifications which are authorized for use but not completely closed will be reported based on the date of their final closure. For ease of reference, the changes have been identified by their design change control modification number.

Modification No.	Description
	HPCI Drain Pot Line Supports
M12-2-89-19 and M12-3-89-19	This modification involved the installation of improved seismic supports for the High Pressure Coolant Injection (HPCI) turbine Steam Supply Valve Drain Pot piping. The improved supports provide greater margin in the seismic analyses associated with this piping.

5.6 Temporary System Alterations (Unit 2 and Unit 3)

A "Temporary System Alteration" refers to electrical jumpers, lifted leads, removed fuses, fuses turned to non-conducting position, fuses moved from normal to reserve holder, temporary power supplies, test switches in alternate positions, temporary blank flanges, and spool pieces. Alterations controlled and documented as part of a routine out-of-service or other procedure, alterations which are a normal feature of system design, and hoses installed as part of a venting or draining process are not included.

Temporary system alterations performed during October, 1989 will be reported in the November report.

5.6.1 Unit 2

Temporary System Alteration No.	Description	Installation Date	Removal Date
II-68-89	Alteration to install a temporary 208 VAC feed for a Reactor Cooling Sample Panel Cooling Unit.	10-15-89	-
II-69-89	Installation of a jumper at junction box 2CB-21 in the cribhouse to prevent spurious alarming of the XL3 fire computer until completion of permanent repairs.	10/17/89	10/18/89
II-70-89	Installation of a jumper on the 902-54 panel to prevent a continuous audio alarm.	10/19/89	10/19/89
II-71-89	Alteration to disconnect analog process computer point A2840 (2A torus wide range level) pending installation of a new signal isolator.	10/20/89	-
II-72-89	Alteration to disconnect analog process computer point A2096 (2B torus wide range level) pending installation of a new signal isolator.	10/23/89	-

5.6.1 Unit 2 (Cont'd)

Temporary System Alteration No.	Description	Installation Date	Removal Date
II-73-89	Alteration to disconnect analog process computer point A2850 (wide range drywell pressure) pending installation of a new signal isolator.	10/23/89	-
II-74-89	Alteration to bypass the computer point for the "A" channel of the Main Steam Line radiation monitor during repairs and testing.	10/23/89	10/25/89
II-75-89	Alteration to remove an area radiation monitor indicator/trip unit associated with the vessel instrument rack (station 8, panel 902-11) to facilitate repair.	10/24/89	10/24/89

5.6.3 Unit 3

Temporary System Alteration No.	Description	Installation Date	Removal Date
III-37-89	Installation of a jumper at panel 903-4 to allow for cycling of shutdown cooling (3-1001-2C) heat exchanger inlet valve during maintenance and testing.	10/6/89	10/17/89
III-38-89	Installation of a jumper at panel 903-4 to allow for cycling of shutdown cooling heat exchanger inlet valve (3-1001-2A) during maintenance and testing.	10/14/89	10/14/89
III-39-89	Installation of a temporary 208 VAC feed for a reactor coolant sample panel.	10/15/89	-

5.6.3 Unit 3 (Cont'd)

Temporary System Alteration No.	Description	Installation Date	Removal Date
III-40-89	Installation of a jumper at panel 903-4 to allow for cycling of shutdown cooling heat exchanger inlet valve (3-1001-2C) during maintenance and testing.	10/17/89	10/19/89
III-41-89	Alteration to disconnect analog process computer point A3840 (3A torus wide range level) pending installation of a new signal isolator.	10/23/89	-
III-42-89	Alteration to disconnect analog process computer point A3096 (3B torus wide wide range level) pending installation of a new signal isolator.	10/23/89	-
III-43-89	Installation of a bearing outboard of the existing HPCI room cooler blower shaft to minimize wear of the shaft.	10/23/89	-