



CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

RR#1 • BOX 127E • EAST HAMPTON, CT 06424-9341

October 27, 1989
Re: 10CFR50.73(a)(2)(i)(B)

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. DPR-61
Docket No. 50-213
Reportable Occurrence LER 50-213/89-018-00

Gentlemen:

This letter forwards the Licensee Event Report 89-018-00, required to be submitted, pursuant to the requirements of Connecticut Yankee Technical Specifications.

Very truly yours

Donald B. Miller, Jr.
Station Superintendent

DBM:REB/mlg

Attachment: LER 50-213/89-018-00

cc: Mr. William T. Russell
Regional Administrator, Region I
475 Alienable Road
King of Prussia, PA 19406

J. T. Shedlosky
Sr. Resident Inspector
Haddam Neck

8911020066 891027
PDR ADOCK 05000213
S PDC

Handwritten initials: E22
Handwritten number: 11

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Haddam Neck	DOCKET NUMBER (2) 0 5 0 0 0 2 1 3	PAGE (3) 1 OF 0 3
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TITLE (4)
Containment Penetration Fails Type C Local Leak Rate Test

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 9	2 9	8 9	8 9	0 1	8 0	0 1	0 2	7 8 9			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9) 6	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0	20.402(b)	20.406(c)	50.73(e)(2)(iv)	73.71(b)						
	20.405(a)(1)(i)	50.36(e)(1)	50.73(a)(2)(v)	73.71(e)						
	20.405(a)(1)(ii)	50.36(e)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)							
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)						TELEPHONE NUMBER					
NAME J. L. DeLawrence, Asst. Engineering Supervisor						AREA CODE 2 0 3					
						2 6 7 - 2 5 5 6					

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)						<input type="checkbox"/> NO		0 3	0 1	9 0

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

ABSTRACT

Containment penetration local leak rate testing (Type B and C) is being conducted during the 1989 refueling outage in accordance with 10CFR50 Appendix J and Technical Specification 4.4.II.

On September 29, 1989, at approximately 2215, with the plant shutdown in mode 6, containment penetration P-65 (Air Monitor Sample to Containment) Isolation Valve VS-CV-1104 failed its as found Type C local leak rate test. The root cause of the failure has not been determined. The valve will be removed and inspected and the results of the inspection and corrective action will be forwarded in a supplemental report.

This event is reportable per 10CFR50.73(a)(2)(i)(B) since it involves a condition prohibited by the plant's Technical Specifications.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Haddam Neck	DOCKET NUMBER (2) 0 5 0 0 0 2 1 3	LER NUMBER (6)			PAGE (3)		
		YEAR 8 9	SEQUENTIAL NUMBER - 0 1 8	REVISION NUMBER - 0 0			
					0 2	OF 0 3	

TEXT (If more space is required, use additional NRC Form 388A's) (17)

BACKGROUND INFORMATION

Containment penetration local leak rate testing (Type B and C) is being conducted during the current refueling outage in accordance with 10CFR50 Appendix J and Technical Specification 4.4.II. This testing is used in conjunction with containment (EIIS Code: JM) integrated leak rate testing (Type A) to assure containment integrity.

The Technical Specification acceptance criteria for the allowable leakage from the sum of all penetration and isolation valve local leak rate tests is 0.6La (650 LB mass/day). La is defined as the maximum allowable leakage rate and is equal to 0.18 weight percent of the contained air per 24 hours at a 40 psig test pressure.

EVENT DESCRIPTION

On September 29, 1989, at approximately 2215, with the plant shutdown in mode 6, containment penetration P-65 (Air Monitor Sample to Containment) Isolation Valve VS-CV-1104 failed the as found Type C local leak rate test with a leak rate in excess of 0.6La (650 LB mass/day). VS-CV-1104 is one of two check valves in series. The second check valve tested in P-65 is VS-CV-1103 which passed the local leak rate test with a leak rate of 3.56 LB mass/day.

CAUSE OF THE EVENT

VS-CV-1104 is a 1" Swing Disc Check Valve manufactured by NIBCO. VS-CV-1104 will be removed from the Air Monitor Sample line and inspected to determine the root cause of failure. The results of this inspection will be forwarded in a supplemental report.

SAFETY ASSESSMENT

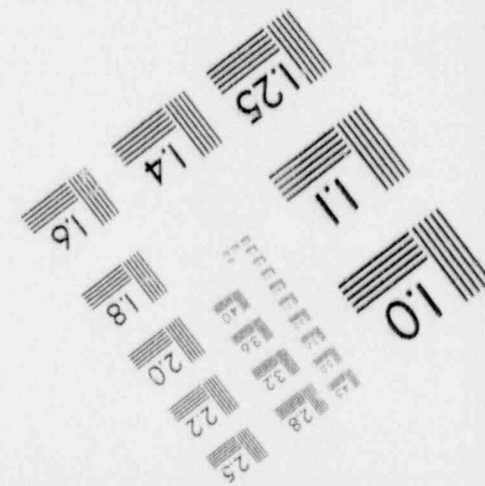
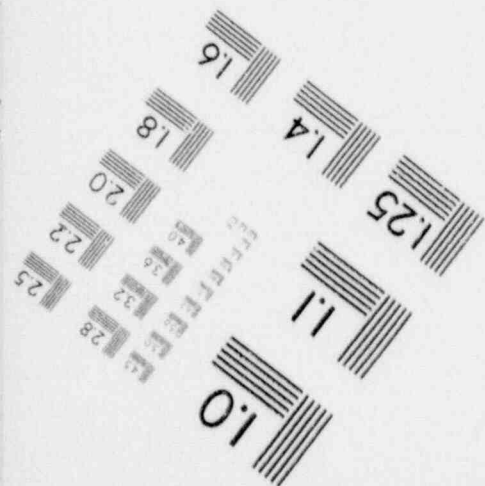
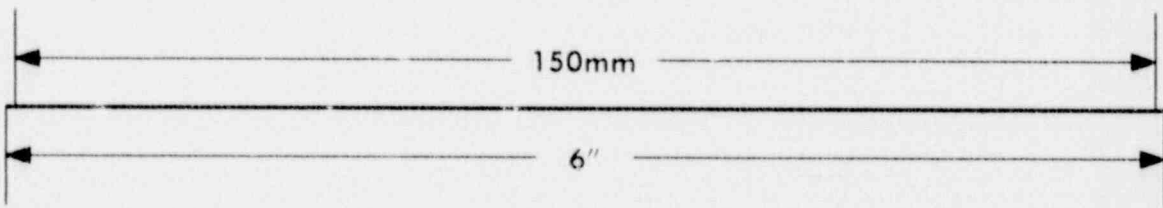
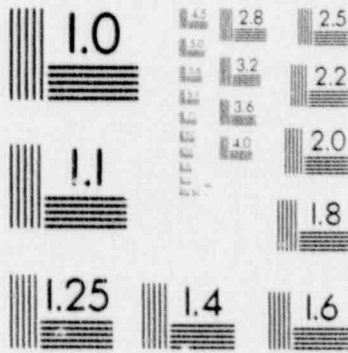
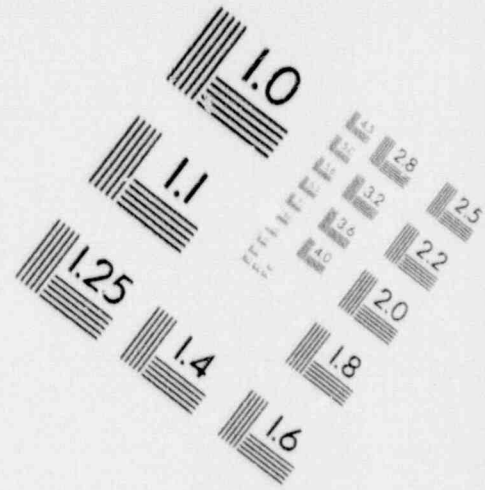
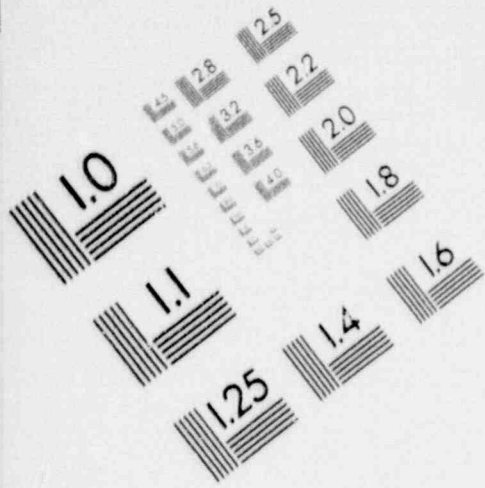
This event is reportable per 10CFR50.73(a)(2)(i)(B) since it involves a condition prohibited by the plant's Technical Specifications.

VS-CV-1104 and VS-CV-1103 are located in series with respect to the containment, and as such, afford redundant protection.

Since these two valves are located in series, the excessive leakage past VS-CV-1104 would be stopped by VS-CV-1103 before it was able to escape downstream of the penetration. Based on this fact, the excessive leakage past VS-CV-1104 is judged to have limited safety significance.

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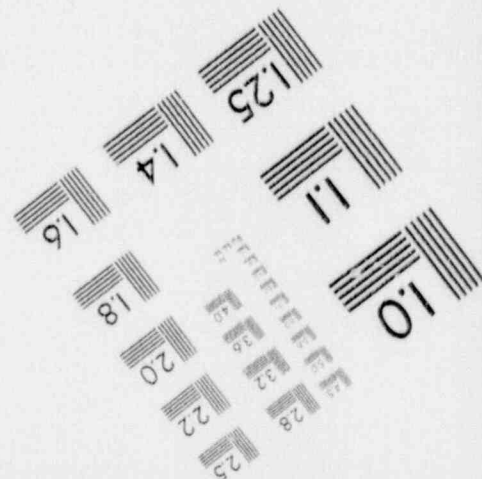
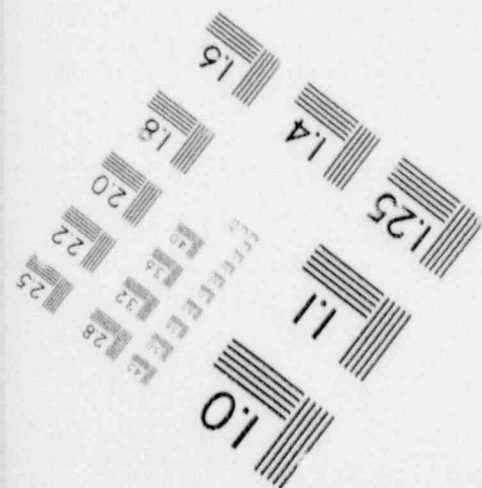
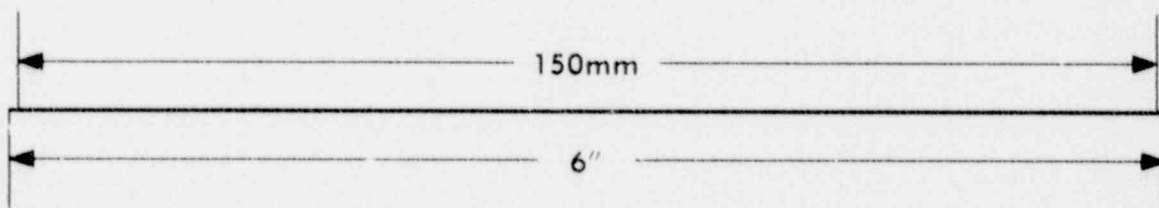
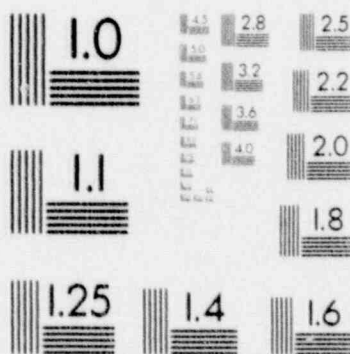
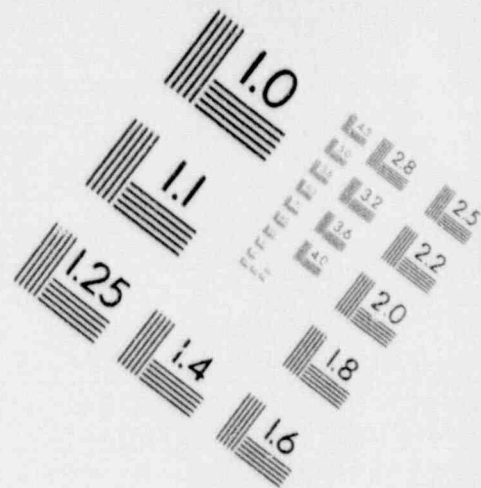
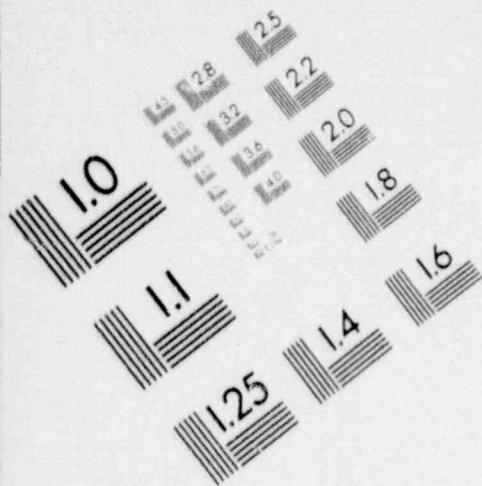
IMAGE EVALUATION TEST TARGET (MT-3)



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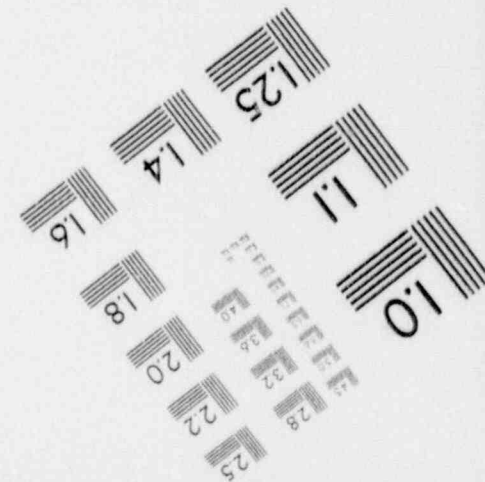
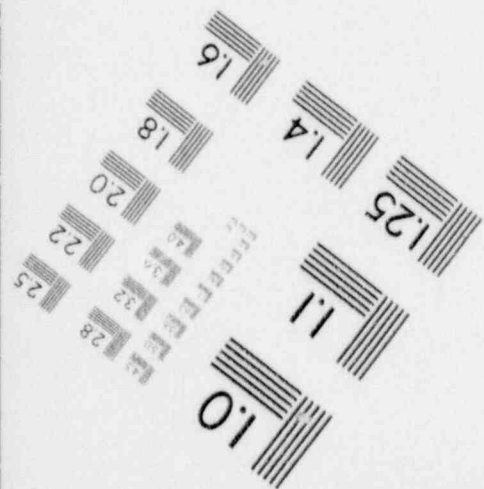
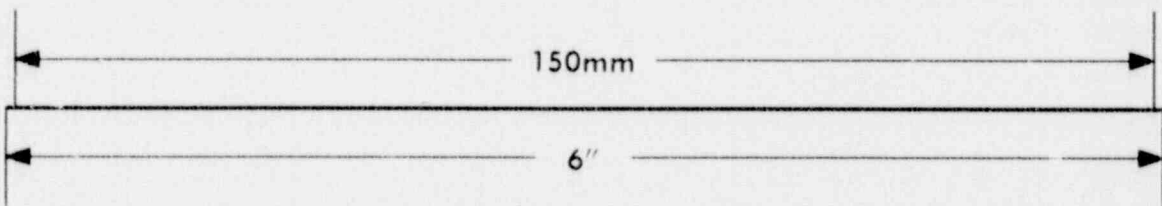
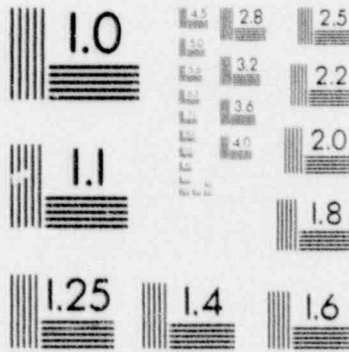
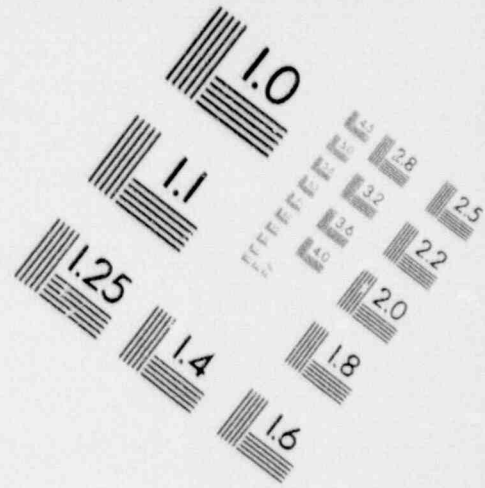
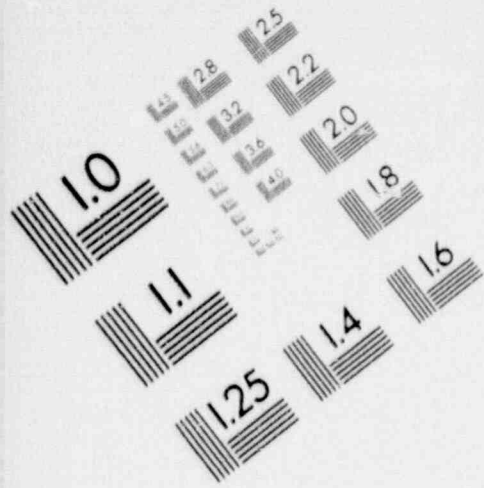
IMAGE EVALUATION TEST TARGET (MT-3)



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IMAGE EVALUATION TEST TARGET (MT-3)



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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Haddam Neck	DOCKET NUMBER (2) 0500021389	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		01	8	00	03	OF	03

TEXT (If more space is required, use additional NRC Form 360A's) (17)

CORRECTIVE ACTION

The valve will be removed from the Containment Air Monitor line and inspected to determine root cause of failure. The results of this inspection will determine corrective action. Root cause and corrective action information will be forwarded in a supplemental report.

ADDITIONAL INFORMATION

VS-CV-1104 is a 1 inch, 200 psi, bronze Swing Check valve manufactured by NIBCO.

PREVIOUS SIMILAR EVENTS

- LER 87-011
- LER 86-006
- LER 84-012
- LER 83-04-L