NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Shoreham Nuclear Power Station Unit #1	0 5 0 0 0 3 2 2	1 OF 0 3

TITLE (4) RBSVS "B" Side Initiation While Performing Maintenance on the Valve Actuator for the RBNVS System Intake Valve

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
1 0	0 6	8 6	8 6	- 0 3 9	- 0 0	1 0	2 4	8 6		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
4	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 1 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Robert W. Grunseich, Operational Compliance Engineer	5 1 1 6 9 2 9 - 1 8 3 1 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	V A	I S V	B 2 3 7	No					

SUPPLEMENTAL REPORT EXPECTED (14)

	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 6, 1986 at 1346 there was an unplanned initiation of the Reactor Building Standby Ventilation System (RBSVS) "B" Train. The plant was in Operational Condition 4 (Cold Shutdown) with the mode switch in Shutdown and all rods inserted in the core. While clearing a tag on valve 1T46*AOV-035B per Station Equipment Clearance Permit 86-10-28, the Equipment Operator (EO) was backing out a jacking screw from the valve actuator. The actuator, pressurized with station air, ejected the screw, as it neared the end of its thread engagement. On loss of air to the actuator, the valve isolated causing the RBSVS initiation. Plant Management was notified and the NRC was notified at 1455 per 10CFR50.72. The jacking screw was then replaced and the Reactor Building Normal Ventilation System was returned to service. Maintenance Work Requests (MWRs) have been generated to accomplish the marking of the jacking screw, and for similar actuators, to indicate the limit to which the screw can be safely withdrawn, and warning signs will be posted at each actuator to indicate the hazard. Engineering will be requested to evaluate the feasibility of a retaining device to eliminate the potential for recurrence of this event. The Incident Report will become required reading for all operators, mechanics, and technicians.

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PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as [xx].

IDENTIFICATION OF THE EVENT

Reactor Building Standby Ventilation System (RBSVS) [VA] "B" initiation while performing maintenance.

Event Date: 10/6/86

Report Date: 10/24/86

CONDITIONS PRIOR TO THE EVENT

Operational Condition 4 (Cold Shutdown)

Mode Switch - Shutdown

RPV Pressure = 0 psig RPV Temperature = 145 degrees F

All rods inserted in the core

DESCRIPTION OF THE EVENT

On October 6, 1986 at 1346 there was an unplanned initiation of the Reactor Building Standby Ventilation System (RBSVS) "B" Train. The Equipment Operator (EO) was clearing a tag on Reactor Building Normal Ventilation System (RBSVS) valve 1T46*AOV-035B as required by Station Equipment Clearance Permit (SECP) 86-10-28. The jacking screw on the valve actuator had been fully inserted to block the system intake isolation valve open and allow I&C to repair an instrument air leak on the actuator. The EO was backing the jacking screw out when air pressure to the actuator ejected the screw and loss of air caused the valve to isolate. In response to the isolation signal, the RBSVS "B" Train initiated. Plant Management was notified at 1456 and the NRC was notified at 1455 per 10CFR50.72. The jacking screw was re-installed and the RBSVS was returned to service. The RBSVS was in service for approximately one (1) hour prior to being secured.

CAUSE OF THE EVENT

The cause of the incident was that the design did not require a retaining device or some indication that the screw was nearing the end of its thread engagement. Nor was it specified in any Station Procedure that the screw was part of the pressure boundary of the actuator. The EC involved in this incident is qualified per SP21.006.01.

ANALYSIS OF THE EVENT

The event was an unplanned actuation of an Engineered Safety Feature (RBSVS), and is reportable per 10CFR50.73(a)(2)(iv). There was minimal safety significance to the event. The RBSVS initiated on an isolation signal per design. Had the event occurred under a more severe set of circumstances (5% power) there would be no appreciable increase in significance.

CORRECTIVE ACTIONS

Twelve Maintenance Work Requests have been initiated, MWR's 86-5008 through 86-5019, inclusive, to accomplish the marking of the jacking screw on this and similar valves to the limit the screw can safely be withdrawn, and warning signs will be affixed to each actuator to delineate the hazard, consequences and the reason for the markings. Engineering will be requested to evaluate the feasibility of a permanent retaining device to eliminate the potential for recurrence of this event. To prevent recurrence, the Incident Report, IR 86-022, will become required reading for all operators, mechanics, and technicians.

ADDITIONAL INFORMATION

a. Manufacturer and model number of failed component (s)
Fisher Controls Company - 72" Type 9220 Valve Assembly with Bettes T-316 SR2-M3 Rotary Actuator.

b. LER numbers of previous similar events
None

LILCO

LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION • P.O. BOX 628 • WADING RIVER, NEW YORK 11792

TEL. (516) 929 8300

October 24, 1986 PM-86-285

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

In accordance with 10CFR50.73, enclosed are copies of Shoreham's Nuclear Power Station Unit 1's Licensee Event Reports 86-038 and 86-033.

Sincerely yours,

William E. Steiger, Jr.
Plant Manager

WES/pz

Enclosure

cc: Dr. Thomas [illegible]ley, Regional Administrator
John Berry, [illegible]ior Resident Inspector
Institute of [illegible]ear Power Operations, Records Center
American Nuclear Insurers

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