



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

October 16, 1989

Docket No. 50-395

LICENSEE: South Carolina Electric & Gas Company  
FACILITY: V. C. Summer Nuclear Station, Unit No. 1  
SUBJECT: SUMMARY OF SEPTEMBER 25, 1989 MEETING WITH SOUTH CAROLINA  
ELECTRIC & GAS COMPANY

GENERAL

On September 25, 1989, representatives of the Office of Nuclear Reactor Regulation (NRR) met with representatives of the South Carolina Electric & Gas Company (SCE&G) to discuss the status of the licensing issues for the V. C. Summer Nuclear Station. The meeting was held at the NRR offices in Rockville, Maryland. A list of those persons who attended the meeting is included as Enclosure 1.

DISCUSSION

A handout was provided at the meeting which presented the NRC's current assessment of the status of various licensing actions. This handout is included as Enclosure 2. There was an error in the title of the TAC 71554 listed on Enclosure 2. It should have read "GL 88-11, NRC Position on Radiation Embrittlement of Reactor Vessel Materials and its Impact on Plant Operations." Enclosure 3 is a handout provided by the licensee which listed proposed technical specification changes needed for the next Refueling Outage and other technical specification changes to be requested.

Enclosure 4 is a letter from the Westinghouse Electric Corporation to the Westinghouse Owners Group Primary Representatives and the Tech Spec Subcommittee Representatives which included the NRC's Safety Evaluation (SE) on WCAP-11736, "Residual Heat Removal System Autoclosure Interlock (ACI) Removal Report". This letter was also provided during the meeting. The SE approved the WCAP for reference provided that five key improvements were made. Enclosure 5 is a handout provided by the licensee during the meeting which indicated on the licensee's submittal for removal of the RHR System ACI at the Summer Station how the Summer Station was affected by two of the required improvements.

Discussions were also held on the amendment submittals involving F\*, L\*, and the steam generator tube sleeves. As a result of the meeting a conference call was scheduled for October 3, 1989 to discuss the F\* amendment request and a meeting was scheduled for October 5, 1989 to discuss the licensee's strategy for addressing steam generator problems at the Summer Station during the next refueling outage scheduled for March 1990.

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The next meeting is currently scheduled for November 1989, in Region II offices in Atlanta, Georgia.

Original Signed By:

John J. Hayes Jr., Project Manager  
Project Directorate II-1  
Division of Reactor Projects - 1/II  
Office of Nuclear Reactor Regulation

Enclosures:  
As stated

cc w/encls:  
See next page

|      |              |            |   |   |   |   |   |   |
|------|--------------|------------|---|---|---|---|---|---|
| OFC  | : PDII-1     | : PDII-1   | : | : | : | : | : | : |
| NAME | : JHayes: sw | EAdensam   | : | : | : | : | : | : |
| DATE | : 10/16/89   | : 10/16/89 | : | : | : | : | : | : |

OFFICIAL RECORD COPY  
Document Name: (Summer mtg. summary)



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South Carolina Electric & Gas Company

Virgil C. Summer Nuclear Station

cc:

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Mr. A. R. Koon, Jr., Manager  
Nuclear Licensing  
Virgil C. Summer Nuclear Station  
P. O. Box 88  
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ATTENDANCE

September 25, 1989

SCE&G

A. Koon, Jr.  
K. Nettles

NRC

H. Conrad  
J. Hayes

## STATUS OF NRC TACS FOR THE V. C. SUMMER PLANT

September 25, 1989

49976 ISI PROGRAM (M. HUME)

-ITEM NOW GIVEN LOW PRIORITY.

51287 SPDS (G. WEST)

-SER ISSUED

-THREE OPEN ITEMS

1. TOP LEVEL ICONICS ARE NOT CONTINUOUSLY DISPLAYED.

2.

a. SYSTEM RESPONSE TO OPERATOR COMMANDS IS VERY SLOW.

b. SYSTEM HAS NOT BEEN ABLE TO MEET ITS UNAVAILABILITY GOAL OF LESS THAN 1 %.

c. LACK OF REDUNDANCY IN THE DATA ACQUISITION CONTROLLER AND THE UNIBAR CONTROL SWITCH INCREASES THE VULNERABILITY OF THE SYSTEM TO FAILURES CAUSED BY THESE COMPONENTS.

3. HUMAN FACTOR PRINCIPALS NEED TO BE RESOLVED.

4. SOME NON-VALID VALUES OR CONDITIONAL FLAGS EXISTED FOR RHR FLOW RATE, SUBCOOLING MARGIN, REACTOR VESSEL LEVEL INDICATION SYSTEM (RVLIS), STARTUP RATE, CONTAINMENT HYDROGEN, AND SAFETY INJECTION (SI) FLOW.

-RESPONSE SENT FROM THE LICENSEE 6/16/88.

-LICENSEE COMMITTED TO A NEW SPDS TO BE INSTALLED DURING 1990 REFUELING OUTAGE.

-WILL NECESSITATE NEW NRC REVIEW CYCLE.

-ANTICIPATED CLOSURE OF THIS ITEM IS 12/31/90.

-LETTER SENT TO SCE&amp;G 11/22/88 INDICATING REVISED REVIEW SCHEDULE.

-LICENSEE PROVIDED A MILESTONE SCHEDULE FOR THE INSTALLATION OF THE SPDS ON 3/6/89.

-TAC CLOSED 7/17/89. NOW COVERED BY GL 89-06 (TAC 73713)

51606 RV AND SV TESTING (II.D.1) (GARY HAMMER)

-SCE&amp;G SUBMITTED RESPONSE TO STAFF COMMENTS ON 5/9/88.



- ADDITIONAL QUESTIONS SUPPLIED TO SCE&G AS A RESULT OF THEIR 5/9/88 RESPONSE.
- DRAFT OF SCE&G RESPONSES TO NRC QUESTIONS GIVEN TO HAMMER ON 7/25/88.
- CONFERENCE CALL HELD ON THE RESPONSES 8/10/88.
- SCE&G TRANSMITTED FOLLOWING INFORMATION IN DRAFT FORM:
  - (1) DRAWING C-314-601 WENT TO EG&G, IDAHO 8/11/88;
  - (2) ACCEPTANCE CRITERIA FOR NORMAL, EMERGENCY, AND UPSET CONDITIONS ON 9/21/88 TO C HAMMER;
  - (3) EXPLANATION OF DISCREPANCIES BETWEEN RESPONSES TO QUESTIONS 6 AND 9 AND FSAR TABLE 5.2.3 ON 9/21/88 TO G. HAMMER.
- NRC PROVIDED COMMENTS ON RESPONSE TO ITEM B OF SCE&G DRAFT RESPONSES.
- SCE&G PROVIDED RESPONSES TO NRC COMMENTS ON 5/31/89.

53720 ITEM 2.2 OF GL 83-28, EQUIP. CLASSIFICATION AND VENDOR INTERFACE FOR SAFETY RELATED COMPONENTS (DON LASHER)

- FARLEY TO BE HANDLED AS THE LEAD PLANT
- TER ISSUED BY EG&G
- OPEN ITEMS ARE:
  - CONTACT VENDORS OF KEY COMPONENTS SUCH AS AFW PUMPS, ECCS PUMPS SAFETY RELATED BATTERIES, AND SAFETY RELATED VALVES TO FACILITATE THE EXCHANGE OF CURRENT TECHNICAL INFORMATION. AN INFORMAL INTERFACE AGREEMENT SHOULD BE ESTABLISHED.
  - FOR DGs AND SAFETY RELATED SWITCHGEAR VENDORS A FORMAL INTERFACE SIMILAR TO THAT WITH THE NSSS SHOULD BE ESTABLISHED.
- GENERIC LETTER TO BE ISSUED LATE SUMMER.
- GENERIC LETTER WILL PROVIDE GUIDANCE ON VENDOR INTERFACE, LICENSEE WILL BE REQUESTED TO CONFIRM BY LETTER THAT THEY MEET GL, AND NRC MAY INSPECT LICENSEE'S PROGRAM.

53942 ITEMS 4.2.3 & 4.2.4 OF GL 83-28, PREVENTIVE MAINTENANCE PROGRAM FOR RX TRIP BREAKERS, LIFE TESTING & REPLACEMENT (C. MORRIS)

- BULLETIN IS BEING PREPARED TO SEND TO CRGR.

55344 PROCEDURES GENERATION PACKAGES (SUPPLEMENT 1, NUREG- C737) (GREG GALLETTI)

- DRAFT SE ISSUED 6/29/87.
- PROBLEMS IDENTIFIED INCLUDED:
  1. PLANT SPECIFIC TECHNICAL GUIDELINES.

2. WRITERS GUIDE.
  3. PROGRAM USED FOR VALIDATION OF EOPs.
  4. TRAINING PROGRAM FOR UPGRADED EOPs.
- SCE&G WAS TO RESPOND TO THE STAFF COMMENTS WHEN FURTHER STAFF GUIDANCE IS PROVIDED (SEE BELOW)..
  - NRC TO PERFORM INSPECTION OF 16 FACILITIES TO DETERMINE IF A DIFFERENT APPROACH IS NECESSARY IN THIS AREA.
  - NUREG TO BE ISSUED 4/89 TO PROVIDE ADDITIONAL GUIDANCE ON PGP's.
  - SEs TO BE WRITTEN ON EACH PLANT WITH PLANT SPECIFIC COMMENTS ON THE PGP's. LICENSEE IS TO INCORPORATE THE COMMENTS OR MAINTAIN A COPY OF THE JUSTIFICATION AS TO WHY THE COMMENTS WERE NOT INCORPORATED.
  - SUMMER SE TO BE ISSUED BY THE TECHNICAL BRANCH

59402 40 YEAR OL LIFE AMENDMENT

- LICENSEE RESPONDED TO RAI 3/30/88.
- PM HELD MEETING WITH THE LICENSEE THE WEEK OF 4/18-4/21 TO DISCUSS THE ADDITIONAL INFORMATION REQUIRED TO EVALUATE THE AMENDMENT REQUEST.
- LETTER SENT TO LICENSEE REQUESTING ADDITIONAL INFORMATION 4/20/89.
- LICENSEE COMMITTED TO PROVIDING RESPONSE BY 6/19/89 IN 5/31/89 LETTER.
- LICENSEE RESPONSE TRANSMITTED 6/15/89.
- REQUEST FOR ADDITIONAL INFORMATION TRANSMITTED 8/16/89.
- LICENSEE TRANSMITTED A RESPONSE ON 8/28/89.

54821 RSB 5-1, NATURAL CIRCULATION TEST OF THE REACTOR COOLANT

- REACTOR SYSTEMS BRANCH TO REVIEW THIS ITEM.
- CONFERENCE CALLS HELD 6/21/89, 7/19/89 AND 8/29/89.
- SCHEDULE FOR SUBMITTAL BY LICENSEE IS 11/89.

65060 REMOVAL OF FIRE PROTECTION REQUIREMENTS FROM THE TS (GL 86-10)

- RE-SHOLLIED 2/24/88.
- AMENDMENT TO HAVE BEEN ISSUED 5/88.
- CONCURRENCE STOPPED DUE TO FORTHCOMING GUIDANCE IN GENERIC LETTER TO BE ISSUED.
- GENERIC LETTER ISSUED 8/3/88.
- MEETING HELD 8/17/88 TO DISCUSS REVISED SUBMITTAL AND GENERIC LETTER.
- LICENSEE SUBMITTED ADDITIONAL INFORMATION 1/31/89.
- AMENDMENT ISSUED 7/24/89.



- 65778      REMOVAL OF ORGANIZATIONAL CHARTS FROM THE TS
- SCE&G PROVIDED RESPONSE BASED UPON GL 88-06 ON 5/16/88..
  - RE-SHOLLIED 8/25/88.
  - SUBMITTAL RECEIVED FROM SCE&G 12/22/88.
  - SE ISSUED 7/24/89.
- 67340      ORGANIZATIONAL CHARTS TO REFLECT 1/25/88 REORGANIZATION
- SUBMITTAL RECEIVED FROM SCE&G 12/22/88.
  - SE ISSUED 7/24/89.
- 67812      ISI RELIEF, HYDROSTATIC TESTING OF THE SGB SYSTEM (M. HUME)
- RELIEF GRANTED 6/22/89.
- 68046      IST 12/23/87 PROGRAM REVISION (K. DEMPSEY)
- SER ISSUED 4/19/88
  - SOME RELIEFS DENIED, SOME GRANTED
  - LICENSEE MUST ADDRESS APPENDIX C OF THE TER WHICH COVERS OMISSIONS AND INCONSISTENCIES IN THE PROGRAM.
  - MEETING HELD 6/1/88 TO DISCUSS RELIEFS WHICH WERE DENIED AND 12/23/87 REVISED PROGRAM.
  - REVISED PROGRAM SUBMITTED BY SCE&G 8/31/88.
  - ADDITIONAL INFORMATION REQUESTED VIA THE TELEPHONE ON 6/15/89 FOR EG&G TO COMPLETE THE REVIEW.
  - PROCEDURE GTP 302 TRANSMITTED BY LICENSEE 7/19/89.
- 68610      IMPLEMENTATION OF STATION BLACKOUT RULE, 10 CFR 50.63, A-22 (A. GILL)
- RESPONSE TRANSMITTED 4/17/89.
- 68834      BULLETIN 88-05, NON-CONFORMING MATERIALS SUPPLIED BY PIPING SUPPLIES, INC. AT FOLSOM, N.J. AND WEST JERSEY MANUFACTURING CO. AT WILLIAMSTOWN, N. J., B-104.
- RESPONSE SUBMITTED 9/13/88.
  - WRITTEN RESPONSE DUE 60 DAYS AFTER COMPLETION OF ALL SCHEDULED ACTIONS.
  - INSPECTION CONDUCTED BY SCE&G 10/88.
  - LICENSEE SENT LETTER ON RESULTS OF INSPECTION OF NOZZLE PLATES ON 3/6/89.



69066 STEAM GENERATOR, L\* (H. CONRAD)

- ENVIRONMENTAL ASSESSMENT PUBLISHED IN THE FEDERAL REGISTER 10/88.
- CONFERENCE CALL HELD 10/25/88.
- PRELIMINARY REVIEW INDICATES THAT ADDITIONAL INFORMATION IS REQUIRED.
- TECHNICAL STAFF REVIEW TO BEGIN UPON RECEIPT OF ADDITIONAL SCE&G INFORMATION.
- ADDITIONAL INFORMATION TRANSMITTED 6/13/89.

69693 BULLETIN 88-08, "THERMAL STRESSES IN PIPING CONNECTED TO REACTOR COOLANT SYSTEMS

- RESPONSE SUBMITTED 9/29/88.
- MEETING HELD 10/6/88.
- ADDITIONAL INFORMATION SUBMITTED BY SCF&G 12/19/88.
- DIFFERENTIAL TEMPERATURE DATA SUBMITTED ON 3/23/89.

69782 GENERIC LETTER 88-17, "LOSS OF DECAY HEAT REMOVAL"

- RESPONSE TO EXPEDITIOUS ACTIONS PROVIDED BY SCF&G 1/23/89.
- RESPONSE TO 6 PROGRAM ENHANCEMENTS PROVIDED BY LICENSEE 2/2/89.
- NRC COMMENTS ON EXPEDITIOUS ACTIONS SENT TO LICENSEE 6/26/89.

69978 BULLETIN 88-04, POTENTIAL SAFETY RELATED PUMP LOSS.

- WRITTEN RESPONSE DATED 7/8/88.
- RESPONSE ON EFW PUMPS PROVIDED 12/20/88.
- RESPONSE REQUIRED ON WHETHER INSTALLED MINIFLOW CAPACITY IS ADEQUATE FOR SINGLE PUMP OPERATION. INFORMATION TO BE PROVIDED FOLLOWING 1990 REFUELING OUTAGE.

71360 NRC BULLETIN 88-10, "NON-CONFORMING MOLDED-CASED CIRCUIT BREAKERS".

- LETTER SENT TO SCE&G REQUESTING THAT INFORMATION BE PROVIDED AS IT BECOMES AVAILABLE.
- RESPONSE TRANSMITTED 3/31/89.
- SUPPLEMENT 1 OF THE BULLETIN ISSUED 8/3/89.
- BY APPROXIMATELY 11/10/89 LICENSEE MUST VERIFY PREVIOUS RESPONSE TO THE BULLETIN IN RESPONSE TO SUPPLEMENT 1.

- 71725      GENERIC LETTER 88-14, "INSTRUMENT AIR SUPPLY SYSTEM PROBLEMS AFFECTING SAFETY-RELATED EQUIPMENT"
- RESPONSE SENT 2/2/89.
  - AFFECTED VALVES AND COMPONENTS TO BE ADDED TO THE PREVENTATIVE MAINTENANCE PROGRAM BY 4/28/89.
  - PERIODIC DEWPOINT MONITORING AND SYSTEM BLOWDOWN AT SELECTED LOCATIONS INITIATED BY 8/1/89.
  - FULL IMPLEMENTATION OF A QUALITY RELATED PROGRAM FOR DIAPHRAGMS BY THE FIFTH REFUELING OUTAGE (4/90).
  - LETTER SENT TO LICENSEE ON 3/31/89 REQUESTING THAT THE LICENSEE INFORM THE NRC WHEN ALL REQUIREMENTS ARE IMPLEMENTED.
- 72171      NRC BULLETIN 88-11, "PRESSURIZER SURGE LINE THERMAL STRATIFICATION"
- LICENSEE RESPONDED 3/1/89.
  - SUBMITTED RESPONSE TO 1.A OF BULLETIN.
  - LICENSEE INTENDS TO WORK WITHIN WOG.
  - ALTERNATE SCHEDULE PROPOSED FOR 1.B OF BULLETIN.
  - LETTER SENT 6/12/89 DETAILING AGREEMENT BETWEEN NRC AND THE WESTINGHOUSE OWNERS GROUP. LICENSEES ARE TO SUBMIT THEIR RESPONSE TO ITEM 1.b OF THE BULLETIN WITHIN 1 YEAR AS ORIGINALLY REQUIRED.
- 72727      AMENDMENT ON DC SOURCES (TS 3/4.8.2)
- APPLICATION SENT 8/24/88.
  - SHOLLY ISSUED 5/3/89.
  - AMENDMENT ISSUED 8/7/89.
- 72893      FEEDWATER ISOLATION VALVES (TS 3.7.1.6)
- REQUEST SUBMITTED 4/5/89.
  - SHOLLIED 5/3/89.
- 73209      FAILURE OF WESTINGHOUSE STEAM GENERATOR TUBE MECHANICAL PLUGS (BULLETIN 89-01)
- RESPONSE TO ACTIONS 1-4 ON THE HEATS OF PLUGS IN THE SG SENT 6/12/89.
  - CLOSEOUT ISSUED 9/18/89.
- 73300      SERVICE WATER SYSTEM BIOCIDES TREATMENT
- REQUEST FOR APPROVAL SUBMITTED 5/19/89.
  - CONFERENCE CALL 9/18/89 REQUESTED BETZ SUBMITTAL TO EPA FOR



REGISTRATION OF THE BIOCIDES AND THE EPA RESPONSE BACK TO THAT REQUEST.

- 73301 TS 4.0.3 AND 4.0.4 AMENDMENT.
- REQUEST SUBMITTED 5/22/89.
  - SHOLLIED 7/12/89.
  - AMENDMENT ISSUED 8/8/89.
- 73548 GENERIC LETTER 89-08, "EROSION/CORROSION INDUCED PIPE WALL THINNING
- REPLY REQUIRED BY 7/2/89 AS TO WHETHER A LONG TERM MONITORING PROGRAM HAS BEEN IMPLEMENTED EQUIVALENT TO THE NUMARC. IF NOT, THEN SCHEDULE FOR IMPLEMENTATION.
  - 6/26/89 LICENSEE RESPONSE CONFIRMED ADDRESSING ALL ACTION OF THE GL.
  - GENERAL TEST PROCEDURE 308 TO BE REPLACED BY THE NEW ENGINEERING SERVICE PROCEDURE PRIOR TO THE 5TH REFUELING OUTAGE.
  - LETTER CLOSING THIS OUT TRANSMITTED 7/20/89.
- 73713 GENERIC LETTER 89-06, "TASK ACTION PLAN ITEM I.D.2 - SPDS".
- RESPONSE TRANSMITTED 7/7/89.
  - LICENSEE TO TRANSMIT DESCRIPTION OF REVISED PROGRAM FOR OPERATION FOLLOWING MARCH 1990 REFUELING OUTAGE.
- 74068 GENERIC LETTER 89-13, "SERVICE WATER SYSTEM PROBLEMS AFFECTING SAFETY RELATED EQUIPMENT".
- RESPONSE DUE BY 2/1/90 INDICATING THAT STEPS 1-5 OF THE GL HAVE BEEN IMPLEMENTED.
- 74318 BULLETIN 89-02, "STRESS CORROSION CRACKING OF HIGH HARDNESS TYPE 410 STAINLESS STEEL INTERNAL PRELOADED BOLTING IN ANCHOR DARTLING MODEL 350 W SWING CHECK VALVES OR VALVES OF SIMILAR DESIGN".
- INSPECTION OF CHECK VALVES DUE AT THE NEXT OUTAGE OF DURATION GREATER THAN 4 WEEKS.
  - RESULTS OF INSPECTION DUE WITHIN 60 DAYS OF THE COMPLETION OF THE INSPECTION.
- 74475 GENERIC LETTER 88-20, "INDIVIDUAL PLANT EXAMINATION FOR SEVERE ACCIDENT VULNERABILITY".
- RESPONSE DUE DUE 60 DAYS FROM FEDERAL REGISTER NOTICE ON AVAILABILITY OF NUREG-1335.



- 74559 RELIEF REQUEST ON MAIN STEAM DRAIN VALVES, ASME SECTION XI.  
-RELIEF TO BE ISSUED 10/89.
- 74577 F\* EXTENSION
- 74578 LOSS OF OFFSITE POWER EVENT 7/11/89  
-REQUEST FOR ADDITIONAL INFORMATION REQUESTED 9/6/89.  
-INFORMATION TO BE TRANSMITTED WITHIN 60 DAYS OF THE RECEIPT OF THE REQUEST.
- 74735 GENERIC LETTER 89-07, "POWER REACTOR SAFEGUARDS CONTINGENCY PLANNING FOR SURFACE VEHICLE BOMBS".  
-SUPPLEMENT 1 ISSUED 8/21/89.  
-RESPONSE TO GL REQUIRED APPROXIMATELY 10/28/89.
- 74821 RENUMBERING OF 3/4.7.11
- 74822 TS 3.7.1.5 MSIV MODES 2/3
- 74823 TS SR 4.8.1.1.2.C DG FUEL OIL SAMPLING, ANALYSIS, ETC.
- 74824 TS 3/4.5.2 REMOVAL OF RHR SYSTEM AUTOCLOSURE INTERLOCK (ACI),
- 74825 TS 3.8.1 & 3.8.2 DG FUEL OIL STORAGE VOLUME
- 74826 TS TABLES 2.2-1 & 3.3-4, REPLACEMENT OF RTCs FOR RID BYPASS MANIFOLD
- ????? TS 4.4.5.4 SG TUBE SLEEVES, B&W
- ????? TS DELETION OF CYCLE SPECIFIC PARAMETERS
- ????? BULLETIN 87-02, SUPPLEMENT 1, "FASTENERS TESTING TO DETERMINE CONFORMANCE APPLICABLE MATERIAL SPECIFICATIONS"  
-RESPONSE SUBMITTED 7/20/83.

TACS CLOSED

| <u>TAC NO.</u> | DESCRIPTION OF THE TAC   | DATE CLOSED |
|----------------|--|-------------|
| 53406          | AUTO SHUNT TRIP TS, GL 83-28 ITEM 4.3  | 06/06/89    |
| 54030          | ITEMS 4.5.2 & 4.5.3 OF GL 83-28,<br>REACTOR TRIP SYSTEM FUNCTIONAL TESTING,<br>ALTERNATIVES & TEST INTERVALS | 06/12/89    |
| 59451          | DIESEL GENERATORS TS   | 05/30/89    |
| 56935          | KCP TRIP CRITERIA, GL 83-10D   | 03/15/89    |
| 62803          | CONTAINMENT STRUCTURAL INTEGRITY   | 04/28/89    |
| 66258          | LOSS OF RHR WHILE RCS PARTIALLY FILLED,<br>1/2 LOOP OPERATION.   | 10/17/88    |
| 67327          | RAPIDLY PROPAGATING FATIGUE CRACKS IN<br>SG TUBES, BULLETIN 88-02.   | 11/10/88    |
| 67811          | VANTAGE 5 FUEL, SPENT FUEL STORAGE.  | 10/28/88    |
| 66045          | SINGLE CELL CHARGING OF SAFETY RELATED<br>BATTERIES.   | 04/03/89    |
| 68274          | SG TUBE SLEEVING.  | 11/10/88    |
| 68644          | VANTAGE 5 FUEL, REMAINING TS SECTIONS.   | 10/28/88    |
| 68956          | BORIC ACID CORROSION OF C-STEEL<br>COMPONENTS IN PWR PLANTS GL 88-05.  | 10/06/88    |
| 71554          | NRC BULLETIN 88-10, "NON-CONFORMING<br>MOLDED-CASE CIRCUIT BREAKERS".  | 02/07/89    |
| 72685          | BULLETIN 88-09, "THIMBLE TUBE THINNING<br>IN WESTINGHOUSE REACTORS".   | 01/04/89    |
| 73941          | BULLETIN 88-03, INADEQUATE LATCH<br>ENGAGEMENT IN HFA TYPE LATCHING RELAYS<br>MANUFACTURED BY GE.            | 02/06/89    |

TECHNICAL SPECIFICATION CHANGES NEEDED FOR REFUEL V\* Steam Generator F\* Renewal

Submittal made July 24, 1989 This change provides the ability to not have to unnecessarily plug S/G tubes in the tube sheet area that have cracks within the accepted criteria.

\* Steam Generator L\*

This proposed change expands the accepted criteria of F\* to include additional tubes.

\* Steam Generator Slewing

This proposed change allows the use of a B&W designed "Short Sleeve."

\* RTD Bypass Elimination

This proposed change permits the removal of the RTD Bypass manifold from the Reactor Coolant System.

\* Removal of RHR System Autoclosure Interlock

This proposed change allows for the elimination of the protective interlocks which close the RHR Suction Isolation Valves on high pressure.

\* Removal of Cycle Specific Parameters From Tech. Spec.

This proposed change removes cycle specific parameters from the Technical Specification and be controlled by the Licensee. Part of this change includes the repositioning of Control Rods to address control rod wear.

\* Pressurizer and Main Steam Safety Valve Tolerances +

This proposed change will expand the allowable setpoint tolerances from 11% to -1, + 3% tolerance.

\* RHR Flow During Half-Pipe Operation +

This change will revise the Technical Specification flow valve in regard to Generic Letter 88-17.

+ These changes are to be submitted by October 20, 1989.



## OTHER TECHNICAL SPECIFICATION CHANGES

\* Diesel Generator Fuel Oil Volume

This change updates the Technical Specification for the required volumes.

\* Diesel Generator Fuel Oil Analysis

This change updates the Technical Specification for D/G fuel oil monitoring and analysis to comply with the most current industry standards.

\* Renumbering of Technical Specification 3.7.11

This change corrects a Technical Specification numbering problem subsequent to the deletion of Fire Protection Requirements from Technical Specification.

\* Stroke Time Requirements for Main Steam Isolation Valves & Feedwater Isolation Valves +

This change revised the stroke time requirements for additional margin. (to be submitted by 10/20/89).

\* Reactor Trip System & Engineered Safety Systems Instrumentation Surveillance Interval Relaxation +

This change relaxes the required surveillance intervals in accordance with WOG efforts (to be submitted by 12/31/89).

\* Generic Letter 89-14, "Removal of the 3.25 Limit on Extending Surveillance Interval." +

This change deletes the 3.25 limit on extending surveillance intervals. + (to be submitted by 10/20/89)

+ These changes not yet submitted.



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Terry Wessner - 4  
Wes Higgins  
~~Wes Higgins~~  
JC LaBorde -  
File

WOG-89-160

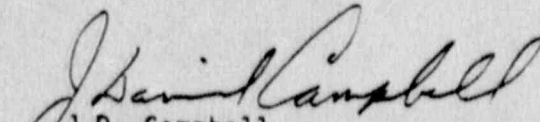
August 28, 1989

To: Westinghouse Owners Group Primary Representatives (1L, 1E)  
Tech Spec Subcommittee Representatives (1L, 1E)

Subject: Westinghouse Owners Group  
RHR ACI Deletion - Safety Evaluation Report (SER)

Enclosed for your information and use is one copy of the Safety Evaluation Report (SER) issued by the NRC letter of August 8, 1989 to Mr. Roger A. Newton from Ashok Thadani for the acceptance of referencing WCAP-11736 "Residual Heat Removal System Autoclosure Interlock (ACI) Removal Report" in plant specific submittals. Efforts are underway to prepare the accepted versions of the WCAP. Note: The original project authorization for this program did not include NRC review fees in the scope. As of December 31, 1988 the NRC review fees were \$19,654. The balance of the review fees for 1989 are unknown and based on current NRC practice we do not anticipate receiving the final invoice until late this year.

Very truly yours,

  
J.D. Campbell  
Project Engineer  
Westinghouse Owners Group

JDC/dac

enclosure

cc: Steering Committee (1L, 1E)  
J.B. George (1L, 1E)  
J.A. Triggiani (1L)