

Thomas W Elward Plant Manager

Big Rock Point Nuclear Plant, 10269 US-31 North, Charlevoix, MI 49720

October 2, 1989

Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Dear Sir:

Enclosed please find the statistical data for the Big Rock Point Nuclear Plant covering the period from Septmeber 1, 1989 through September 30, 1989.

Sincerely,

T W Elward Plant Manager

Enclosures

cc: Administrator Region III, Nuclear Regulatory Commission DRHahn, Department of Public Health RCallen, Michigan Public Service Commission SHall, Michigan Department of Labor PDKrippner, American Nuclear Insurers INPO Record Center NRC Resident Inspector Document Control, Big Rock Point, 740/22*35*10 DPHoffman, P24-117B KWBerry, P24-614B File

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NUCLEAR OPERATION DEPARTMENT Unit Shutdowns and Power Reductions

Report Month	Docket Number	Unit	Date	Completed by	Telephone
September, 1989	55-150	Big Rock Point Plant	October 3, 1989	JRJohnston	(616) 547-6537 ext 223

Number	Date	Туре ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	License Event Report Number	System Code	Component Code ⁵	Cause and Corrective Action to Prevent Recurrence
89-09	09/18/89 thru 09/21/89	1	0 hrs	*					The turbine initial pressure regulator was experiencing pressure swings which resulted in the unit being placed on automatic speed control (synchronous governor). After flushing and making adjustments to the IPR mechanism, the unit was returned to its previous power level.

1F = Forced S = Scheduled 2Reason:

A = Equipment Failure (Explain) B = Maintenance of Test

C = Refueling

D = Regulatory Restriction

E = Operator Training and License Examination

F = Administrative

G = Operational Error (Explain) H = Other (Explain)

3Method:

1 = Manual

2 = Manual Scram

3 = Automatic Scram

4 = Other (Explain)

Exhibit G = Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5Exhibit I = Same Source

GREYBOOK OPERATING DATA REPORT

DOCKET NO. 50-155

NOTES

10 / 1 / 89 JR JOHNSTON DATE: 87: PHONE

616-547-6537, EXT 223

UNIT NAME: BIG ROCK POINT NUCLEAR PLANT
REPORTING PERIOD: 9 / 89
LICENSED THERMAL POWER (MMT): 240
NAMEPLATE RATING (GROSS MWE): 75

OPERATING STATUS

DESIGN ELECTRICAL RATING (NET MME):

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MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 71.0 MAXIMUM DEPENDABLE CAPACITY (NET MWE): 67.0 IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS 3 THRU 7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MME):
10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.0	6551.0	232386.0
NUMBER OF HOURS REACTOR WAS CRITICAL	720.0	4883.9	167112.7
REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
HOURS GENERATOR ON-LINE	720.0	4853.1	164261.2
UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
15. GROSS THERMAL ENERGY GENERATED (MMH)	149120.0	933574.0	31130333.0
GROSS ELECTRICAL ENERGY GENERATED (MAH)	47858.0	302148.0	9879631.0
18. NET ELECTRICAL ENERGY GENERATED (MUH)	45351.6	285797.1	¢343477.0
INIT SERVICE FACTOR	100.01	74.1%	70.73
UNIT AVAILABILITY FACTOR	100.0%	74.13	70.7%
UNIT CAPACITY FACTOR (USING MDC NET)	X0.44	65.13	27.72
UNIT CAPACITY FACTOR (USING DER NET)	87.5%	\$6.62	18.33
23. UNIT FORCED OUTAGE RATE	0.01	0.7X	12.73

24. SHUTDOWNS SCHEDULED OVER NEXT & MONTHS (TYPE, GATE, & DURATION OF EACH) :

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP

		59.40
1 2 3 4 5 6 7 8 9	192.96 191.67	59.37
-	193.21	59.52
4	191.58	59.32
5	191.46	E9.07
6	191.71	59.19
7	203.79	62.10
8	203.87	62.38
	205.46	62.78
.0	210.50	64.17
1	208.71	63.63
2	207.79	63.92
4	209.75	64.33
5	212.83	64.77
6	211.12	64.66
7	211.76	64.46
.8	210.00	64.29
.9	207.25	62.85
0	205.96	62.23
1	206.42	62.28
2	213.37 213.92	63.91 64.28
3	215.71	64.57
5	214.87	64.28
6	216.21	64.40
7	216.50	65.18
8	214.29	64.95
9	216.37	64.86
30	215.96	64.58
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Refueling Information Request

- 1. Facility name: Big Rock Point Plant
- 2. Scheduled date for next refueling shutdown: September, 1990
- 3. Scheduled date for restart following shutdown: October, 1990
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No

If yes, explain:

If no, has the reload fuel design and core configuration been reviewed by Plant Safety Review Committee to determine whether any unreviewed safety questions as associated with the core reload (Reference 10 CFR, Section 50.59)? No

If no review has taken place, when is it scheduled? Next outage

- Scheduled date(s) for submittal of proposed licensing action and supporting information:
- 6. Important licensing considerations associated with refueling, eg, new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design new operating procedures:
- Number of fuel assemblies in: core 84; spent fuel storage pool 254; new fuel storage 0.
- 8. Present licensed spent fuel storage capacity: 441

Size of any increase in licensed storage capacity that has been requested or is planned (in number of fuel assemblies): 0

9. Projected date of last refueling that can be discharged to spent fuel pool assuming the present license capacity: 1995