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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

JUL 13 1980

Docket Nos.: 50-369  
and 50-370

Duke Power Company  
ATTN: Mr. William O. Parker, Jr.  
Vice President - Steam Production  
P. O. Box 33189  
422 South Church Street  
Charlotte, North Carolina 28242

Dear Mr. Parker:

SUBJECT: STEAM GENERATOR MODIFICATIONS  
(McGUIRE NUCLEAR STATION, UNITS 1 AND 2)

Your letter of May 23, 1980, stated that no changes to the McGuire steam generator would be made until the outcome of the Westinghouse steam generator tube testing program. Our current position on the need and requirements for these modifications are discussed below.

Inspection Ports

For some forms of steam generator degradation which have occurred, eddy current testing and tube gauging alone are not sufficient to assess and monitor tube support plate degradation. In order to perform adequate assessment and monitoring of these areas, it is necessary to install inspection ports. These ports should be installed just above the upper support plate and between the tubesheet and the lower support plate and in line with the tube lane. At the upper support plate level, at least one, inspection port is required which shall be large enough for visual observation of the tube lane.

Under the as low as reasonably achievable (ALARA) concept, NRC has been requesting that all possible steam generator modifications be made before the start of operations in order to minimize personnel exposure. Although installation prior to initial operation is preferable, it was determined that the potential installation exposure following the first cycle of operation is not significant enough to justify the delay of the initial start-up of the plant to permit the installation of inspection ports. However, since secondary side contamination will increase as the operating time increases, NRC requires that these ports be installed prior to start-up after the first refueling. Accordingly, in the event that the ports have not been installed in the McGuire steam generator before the license has been issued, the license of each unit will reflect this requirement.

Row 1 Steam Generator Tubes

Experience has shown that the small bend radius of the Row 1 tubes in the steam generators of Westinghouse design leads to early onset of cracking. At the present time NRC is not requiring licensees to plug Row 1 tubes prior to start-up

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or issuance of a full power license. Westinghouse has committed (letter from R. M. Anderson to R. H. Vollmer, May 12, 1980) to a program to determine the particular susceptibility of Row 1 tubes to cracking. The program involves removing numerous tubes from the Trojan plant and subjecting them to non-destructive and destructive testing to identify the cause of the cracking and to develop a field inspection method capable of detecting potential leaking tubes. The results of this evaluation are expected to be available in October 1980. We shall review the program results and decide at that time on the necessity to plug the Row 1 tubes.

Sincerely,

B. J. Youngblood, Chief  
Licensing Branch No. 1  
Division of Licensing

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