

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555



June 4, 1980

Mr. I. R. Finfrock, Jr. Vice President - Generation Jersey Central Power & Light Company Madison Avenue at Punch Bowl Road Morristown, New Jersey 07960

Dear Mr. Finfrock:

Enclosed for your review and comment is the "For Comment" edition of NUREG-0619, "BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking". The report provides the staff's resolution of the NRC's Generic Technical Activity A-10, which is an "Unresolved Safety Issue" pursuant to Section 210 of the Energy Reorganization Act of 1974.

The generic study resulted from the inservice discovery of cracking in feedwater nozzles and control rod drive return line nozzles.

NUREG-0619 describes the technical issues, the technical studies and analyses performed by the General Electric Company and the NRC staff, the staff's technical positions based on these studies, and the staff's plans for licensee and applicant implementation of the technical position.

At the completion of the 60-day period, the staff will evaluate the comments received and, if needed, will issue a supplement or revision to NUREG-0619. The target for issuing the supplement or revision is late August 1980.

Your comments should address the proposed implementation schedule as well as the technical content.

Because of the importance of resolving this issue, plants undergoing Systematic Evaluation Program (SEP) review will be required to implement changes in the same time frame as non-SEP plants.

Sincerely.

Dennis M. Crutchfiela, Chaef Operating Reactors Branch #5

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BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking

Resolution of Generic Technical Activity A-10

R. Snaider, Task Manager

Office of Nuclear Reactor Regulation

U.S. Nuclear Regulatory Commission



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