



RELATED TO
UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555



11/21/79

NOV 21 1979

Docket Nos.: 50-269, 50-270, 50-287,
50-302, 50-312, 50-313,
50-346

TO ALL BABCOCK & WILCOX (B&W) OPERATING PLANTS
(EXCEPT THREE MILE ISLAND, UNITS 1 AND 2)

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - SMALL BREAK LOSS-OF-COOLANT ACCIDENT

Mr. C. Michaelson of the Tennessee Valley Authority has identified a number of concerns which were addressed to the Combustion Engineering System 80 design. However, certain of these concerns are considered applicable to all pressurized water reactors. Therefore, enclosed is a request for additional information, containing the concerns which are applicable to B&W designed plants.

Responses to these questions (including commitments and schedule for those problems which require substantial time for resolution) are requested by December 3, 1979, in order for us to incorporate your responses to NUREG-0565 ("Staff Report on Generic Evaluation of Small Break Loss-of-Coolant Accident Behavior for Babcock & Wilcox Operating Plants").

If any additional information is requested concerning this matter, please contact Mr. R. Capra (301) 492-7745.

R. W. Reid, Chief
Operating Reactors Branch #4
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Enclosure:
Request for Additional
Information - SB LOCA

cc w/enclosure:
See next page

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B&W ADDRESSEES - REQUEST FOR ADDITIONAL INFORMATION -SMALL BREAK LOCA

November 21, 1979

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Request for Additional Information:

Small Break Loss-of-Coolant Accident

1. Transitions from solid natural circulation to reflux boiling and back to solid natural circulation may cause slug flow in the hot leg piping. By use of analysis and/or experiment, address the mechanical effects of the induced slug flow on steam generator tubes.
2. How are the HPSI pumps protected against deadheading if the system repressurizes? If operator action is required, show that there is sufficient time for operator response. How is this covered in guidelines for emergency procedures? (Applicable to Davis-Besse 1 only)
3. Evaluate the impact of RCP seal damage and leakage due to loss of seal cooling on loss of offsite power. How long can the RCP seals sustain loss of cooling without damage?