

NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555

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March 9, 1979

Docket No. 50-313

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT NO. 1

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-313

ARKANSAS POWER & LIGHT COMPANY

NOTICE OF GRANTING OF RELIEF FROM ASME SECTION XI INSERVICE INSPECTION (TESTING) REQUIREMENTS

The U. S. Nuclear Regulatory Commission (the Commission) has granted relief from certain requirements of the ASME Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components" to Arkansas Power & Light Company. The relief relates to the inservice inspection (testing) program for the Arkansas Nuclear One, Unit No. 1 (the facility) located in Pope County, Arkansas. The ASME Code requirements are incorporated by reference into the Commission's rules and regulations in 10 CFR Part 50. The relief is effective as of its date of issuance.

The specific relief granted is as follows: (1) Relief from measuring flow to the accuracy required by the ASME Code for Serivce Water System pumps until flow measuring devices are installed; and (2) Relief from the ASME Code requirement concerning volumetric examination of reactor coolant nozzle-to-vessel welds. The Commission determined that these requirements of the ASME Code a e impractical within the limitations of design, geometry and materials of construction of components, because compliance would result in hard-ships or unusual difficulties without a compensating ircrease in the level of quality or safety.

The requests for reliaf comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the letter granting relief and the related Safety Evaluation. Prior public notice of this action was not required since the granting of this relief from ASME Code requirements does not involve a significant hazards consideration.

The Commission has determined that the granting of this relief will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with this action.

For further details with respect to this action, see (1) the requests for relief dated May 10, 1978 and January 10, 1979, and (2) the Commission's letter to the licensee dated March 8, 1979, and (3) the Commission's related Safety Evaluation.

These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Arkansas Polytechnic College, Russellville, Arkansas .

A copy of items (2) and (3) may be obtained upon request accresses.

to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 8th day of March 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief

Operating Reactors Branch #4
Division of Operating Reactors