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ARKANSAS POWER & LIGHT COMPANY

PO BOX 551 LITTLE ROCK ARKANSAS 72203 - (501) 371 - 4000

November 22, 1976

50-313

1-116-13

Director of Nuclear Reactor Regulation ATTN: Mr. Dennis L. Ziemann, Chief Operating Reactors Branch #2 U. S. Nuclear Regulatory Commission Washington, D. C. 20555



Subject: Arkansas Nuclear One-Unit 1

Proposed Technical Specifications

(File: 1511.1)

Gentlemen:

Canadification

Your letter of September 9, 1976, to Mr. J. D. Phillips requested we revise or resubmit our request for a license amendment to be consistent with the enclosed model technical specifications of your letter which are based on Revision 1 of Regulatory Guide 1.83, dated July 1975. Attached find those changes as requested, save those exceptions which are delineated below.

Specification	Reason for Exception
3.4.5	Limiting conditions for operation of the steam generators presently exist in Specification 3.1. Therefore, we see no need for a change until the conversion to Standard Technical Specifications
4.4.5.3	Subsequent inservice inspections should be allowed at intervals of not less than 10 months rather than 12 months in the event a refueling shutdown is scheduled between 10 and 12 months after the previous inservice inspection.
4.4.5.5	Reporting Requirements presently exist in Specification 6.12.3 for abnormal degradation in the reactor coolant pressure boundary and for conditions that require remedial

1.20.47

Specification

Reason for Exception

action or corrective measures to prevent the existence or development of an unsafe condition. Therefore, we see no need to duplicate these requirements. Annual Operating Report requirements have been revised to include the complete results of the steam generator tube inservice inspection. (St. Technical Specification 6.12.3)

4.4.5.6

This specification was not incorporated as it did not pertain to steam generator tubing surveillance. Presently, redundant level instrumentation provides assurance that the steam generator level is maintained within operable limits.

Table 4.4-2

Minor changes have been incorporated to facilitate the course of action required for each set of circumstances.

3.4.6.2

Limiting conditions for operation pertaining to Reactor Coolant System leakage presently exist in Specification 3.1.6. Therefore, we see no need for a change until the conversion to Standard Technical Specifications.

4.4.6.2

Performance of surveillance requirements to demonstrate Reactor Coolant System leakages would presently pose implementation problems. Therefore, this change has not been proposed but will be included in the submittal for convers n to Standard Technical Specifications.

If you should have any questions and/or comments concerning the above, please contact us.

Very truly yours,

and the state of

J. D. Phillips Senior Vice President

JDP:ay