

<u>IS-261 Item</u>	<u>Component</u>	<u>Exception</u>
6.4	Bolting 2Ø	Not Applicable
6.6	Integrally Welded	Not Applicable

- 4.2.3 The structural integrity of the reactor coolant system boundary shall be maintained at the level required by the original acceptance standards throughout the life of the station. Any evidence, as a result of the tests outlined in Table IS-261 of Section XI of the code, that defects have developed or grown, shall be investigated.
- 4.2.4 To assure the structural integrity of the reactor internals throughout the life of the unit, the two sets of main internals bolts (connecting the core barrel to the core support shield and to the lower grid cylinder) shall remain in place and under tension. This will be verified by visual inspection to determine that the welded bolt locking caps remain in place. All locking caps will be inspected after hot functional testing and whenever the internals are removed from the vessel during a refueling or maintenance shutdown. The inspection of those core barrel to lower grid cylinder locking caps not readily visible (due to guide block interference) may be omitted provided the adjacent thermal shield to lower grid cylinder locking caps are inspected. The core barrel to core support shield locking caps will be inspected each refueling shutdown.
- 4.2.5 Sufficient records of each inspection shall be kept to allow comparison and evaluation of future inspections.
- 4.2.6 Complete surface and volumetric examination of the reactor coolant pump flywheels will be conducted coincident with refueling or maintenance shutdowns such that within a 10 year period after start-up all four reactor coolant pump flywheels will be examined.
- 4.2.7 Vessel specimens will be withdrawn according to a schedule which may be modified to coincide with refueling or maintenance shutdowns. As a minimum requirement the withdrawal schedule will comply with ASTM-E-185-70.

Bases

The surveillance program has been developed to comply with Section XI of the ASME Boiler and Pressure Vessel Code Inservice Inspection of Nuclear Reactor Coolant Systems, 1971, including 1971 Winter Addenda edition.

The vessel specimen surveillance program will be based on equivalent exposure years which will assure that the first sample should be removed near the 50^o NDTT shift point and the last sample should be removed near the end of the vessel design life.